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Annex 1 to Govt. Decree No. 118/2011 (VII. 11.)

Nuclear Safety Code

Volume 1

Nuclear safety authority procedures of nuclear facilities

1.1. INTRODUCTION

1.1.1. Purpose of the regulation

1.1.1.0100. The purpose of this regulation is to lay down the nuclear safety authority procedures related to nuclear facilities and the requirements during the associated activities in order to provide safe, state-controlled use of atomic energy.

1.2. NUCLEAR SAFETY LICENSES RELATED TO NUCLEAR FACILITIES

1.2.1. General provisions

1.2.1.0100. The existence of other licenses specified in other laws is a precondition to the validity of the nuclear safety license.

1.2.1.0200. The issued license shall become invalid if the conditions and obligations specified in the license are not fulfilled or, furthermore if the duration specified in the license expires.

1.2.1.0300. The documents referenced and used to compose the documentation submitted as the substantiation of the license application shall be submitted for the request of the nuclear safety authority.

1.2.1.0310. If not the applicant but the licensee of another nuclear facility or radioactive waste repository owns the property or asset management rights of the real estate, the consent of the licensee of the other nuclear facility or the radioactive waste repository shall be attached to the license application.

1.2.1.0400. In the case of nuclear power plants, the construction, commissioning, operation, operation beyond the designed service life, final shutdown and decommissioning of the nuclear facility shall be licensed per nuclear power plant unit. The construction, final shutdown and decommissioning licenses may be requested in one procedure for similar nuclear power plant units if the conditions for issuing the license exist in terms of each unit, however, the nuclear safety authority shall decide separately upon each individual nuclear power plant unit. The existence of licensing conditions shall be demonstrated for each nuclear power plant unit in the license application.

1.2.1.0500. A joint license may be requested and issued in one procedure for all nuclear power plant units in the case of organisational and management modifications, and for similar units of the same nuclear power plant in the case of technical modifications, and modifications of technical and regulatory documents.

1.2.2. Site survey and assessment license, site license

Site survey and assessment license

Scope of license

1.2.2.0100. The nuclear safety authority, by granting the final binding site survey and assessment license shall accept the suitability of the survey and assessment methods used in the site survey and assessment programme and the theoretical considerations and authorise the performance of further investigations required by the site survey and assessment programme license.

1.2.2.0200. The site survey and assessment license shall be effective until the site license becomes final, but until a maximum of 5 years from its issuance date. The validity of the license may be extended for request by a further 5 years, but the applicant shall demonstrate that the conditions of the license still exist.

Content requirements of the license application

1.2.2.0300. In the site survey and assessment license application:

a) the site survey and assessment programmes and the methods and theoretical considerations to be used as parts of the programme shall be described, and

b) it shall be demonstrated that the methods developed for the determination, survey and assessment of the site are suitable to determine the site-related data required by the design and to determine the suitability of the site.

1.2.2.0400. The site survey and assessment programme shall be attached to the application. Guidance on the requirements for the content of the programme shall be included in a regulatory guideline.

Site license

Scope of the license

1.2.2.0500. The nuclear safety authority by granting the site license shall accept the justification of lack of such site characteristics that would exclude the possibility of construction, furthermore the suitability of the conduction of site survey, assessment of data determined based on the site survey and the site-related design data derived from the assessment and the suitability of the site.

1.2.2.0600. The site license shall be valid until the construction license becomes final but until a maximum of 5 years from its issuance data. For request, the validity of the license can be extended twice by a further 5 years each, but the applicant shall demonstrate that the conditions of the license still exist.

Content requirements of the license application

1.2.2.0700. In the site license application

a) it shall be demonstrated that the site characteristics that would exclude the possibility of construction as laid down in Annex 7 do not exist, and

b) the following shall be described:

ba) the implementation of the licensed site survey and assessment programme, and

bb) determination of the site-related design data.

1.2.2.0800. The complex final report describing the results of the site survey and assessment programme shall be attached to the application. As a part of the complex final report or in a separate document the derivation of site characteristics and their substantiation shall be described. Guidance on the requirements for the content of the complex final report shall be included in a regulatory guideline.

1.2.3. Construction license

Scope of license

1.2.3.0100. Activities that can be performed on the basis of the construction license:

a) preparation of the area necessary for the construction of a nuclear facility;

b) construction of the buildings and building structures of the nuclear facility, construction of systems from system components with or without safety classification (manufacture, procurement and installation) according to the design furthermore, the appropriate construction of the entire nuclear facility by the appropriate connection of systems;

c) cleaning and flushing works preparing for the commissioning of systems and system components;

d) performance of such systems, structures and components function tests which may also be performed without fuel assemblies containing nuclear material, and are actually performed, based on the work programme of tests, ensuring that the systems, structures or components affected by the test shall not or cannot be in any interaction with fuel elements already potentially delivered to the nuclear facility; and

e) in the case of a facility for the interim storage of spent fuel, performance of construction activities associated with its extension.

1.2.3.0110. At variance with the provisions of Section 1.2.3.0100 b):

a) a manufacturing license can be applied for equipment with long manufacturing period, subject to a manufacturing license, specified in the Preliminary Safety Analysis Report, agreed with the nuclear safety authority in advance, after the submission of a construction license application, before the construction license becomes final;

b) a building license may be applied for buildings subject to a building permit, specified in the Preliminary Safety Analysis Report, agreed with the nuclear safety authority in advance, after the submission of a construction license application, before the construction license becomes effective and non-appealable.

1.2.3.0120. When assessing the construction license application, the nuclear safety authority shall examine the harmony between the underlying information submitted with the application for the equipment to be manufactured or buildings and building structures to be constructed on the basis of the licenses specified in Section 1.2.3.0110 and the information stated in the Preliminary Safety Analysis Report submitted with the construction license application.

1.2.3.0130. The authority shall not issue the manufacturing license mentioned in Section 1.2.3.0110 a) or the building license mentioned in Section 1.2.3.0110 b) before issuing the construction license.

1.2.3.0200. The construction license shall be effective until the commissioning license becomes final, but for a maximum of 10 years from its date of issue. The license may be extended by request for a further five years, but the requestor shall verify that the conditions of issuing the license are still in existence. In the case of a facility for the interim storage of spent fuel, the construction license shall be in force until the commencement of the commissioning of the last module. Prior to the start of the separate construction phases, the applicant shall justify that the conditions for issuing the license remain valid.

1.2.3.0210.

Content requirements of the license application

1.2.3.0220. It shall be demonstrated in the license application that the nuclear facility described in the license application can be constructed and safely operated on the site provided with the site license under Section 1.2.2.

1.2.3.0221. The concept plan containing the long-term strategy for spent fuel management – starting from removal from the spent fuel pool until final disposal – shall be submitted as part of the license application.

1.2.3.0230. It shall be demonstrated in the license application that the site properties specified during the site assessment and to be taken into account during the design have been fully taken into account, and the facility has appropriate protection against external hazard factors.

1.2.3.0240. A Preliminary Safety Analysis Report shall be attached to the license application, in which it shall be demonstrated that the nuclear safety requirements for the nuclear facility to be established that fall within the scope of the construction licensing procedure are met.

1.2.3.0250. The Preliminary Safety Analysis Report shall demonstrate that if the safety principles and criteria applied during the design are met in the manner described in the license application, then the nuclear facility to be implemented can be safely operated.

1.2.3.0260. A technical substantiation shall be attached to the Preliminary Safety Analysis Report, which meets at least the requirements set for technical designs in Section 9.3.3.0500 of Annex 9. In addition, the facility model according to Item c) of Section 9.3.1.0600 of Annex 9 shall be submitted.

1.2.3.0270. The Preliminary Safety Analysis Report and the underlying documentation shall have a level of detail that enables the authority to ascertain the fulfilment of the requirements without reviewing further documentation.

1.2.3.0280. Minimum content elements to be presented in the Preliminary Safety Analysis Report of the nuclear power plant:

a) Introduction and general overview of the nuclear power plant

aa) General description of the power plant

aaa) Conditions of construction

aab) Site conditions

aac) Main parameters of the primary and secondary circuits

aad) Connection of the nuclear power plant to the national electricity distribution network

aae) Operating conditions of the nuclear power plant

aaf) Overall protection concept of the nuclear power plant

aag) Environmental impacts of the nuclear power plant

ab) Comparative information, comparison with similar facilities

ac) Information regarding construction

aca) Organisational implementation of the construction

acb) Construction schedule

acc) Description of staging area

acd) Effect of the implementation activity in the case of a site that has an already operational nuclear facility

ace) Description how the requirement of Section 9.5.3.0510 of Annex 9 is complied with

acf) Preliminary emergency response plan related to the commissioning and operation period

acg) Description of experience gained during previous construction activities and non-compliances relevant to safety.

ad) Technical information required but not yet available

ae) Uniform marking system

af) List of documents used and referred to

ag) Drawings and other graphic information

aga) Electrical, control and instrumentation schemas

agb) Pipe connection and instrumentation schemas

agc) Other graphic information

ah) Compliance with the orders of the authorities

b) Description of the site

ba) Determination of the site boundaries by EOVS coordinates, its geographical location, size and distribution of the population

bb) Industrial, transport and military facilities near the site

bc) Meteorology

bd) Hydrology

be) Geology, seismology and geotechnics

bf) Bioeffects

bg) External hazards of human origin

bh) Description of the monitoring programme

c) Design and design principles of systems, structures and system components

ca) Compliance with the orders of the authorities

cb) Classification of systems, structures and system components

cc) Effects arising from normal operating conditions and transients

cd) Evaluation of the parameters of the effects of possible accidents

ce) Protection against extreme weather conditions

cf) Protection against fire, explosion and toxic gases

cg) Protection against aircraft crash

- ch)* Protection against flooding
- ci)* Protection against flying objects
- cj)* Protection against dynamic effects arising as a result of a postulated pipe rupture
- ck)* Earthquake resistance
- cl)* Architectural system components included in safety classes
- cm)* Mechanical systems and system components
- cn)* Electrical, control and instrumentation systems and system components
- co)* Qualification of mechanical, electrical and control and instrumentation, and architectural system components
- d)* The reactor
 - da)* Description of the reactor
 - daa)* Design basis
 - dab)* Presentation of the reactor
 - dac)* Materials of the internal structural components of the reactor
 - dad)* Evaluation of the reactor
 - db)* Fuel system
 - dba)* Design basis
 - dbb)* Description of the characteristics of the fuel system
 - dbc)* Materials of the fuel system
 - dbd)* Inspections to be carried out in order to guarantee the characteristics of the fuel system
 - dbe)* Evaluation of the fuel system
- dc)* Nuclear characteristics
 - dca)* Design basis
 - dcb)* Description of nuclear characteristics
 - dcc)* Methods applied for nuclear design
 - dcd)* Checking the reactor physical parameters of fuel charges
 - dce)* Changes made during design
 - dcf)* Evaluation of nuclear characteristics
- dd)* Thermohydraulics
 - dda)* Design basis

ddb) Thermohydraulic parameters of the active core

ddc) Thermohydraulic parameters of the cooling system of the reactor

ddd) Inspections verifying the compliance of the thermohydraulic parameters

dde) Instrumentation requirements

ddf) Evaluation of thermohydraulic parameters

de) Control and safety protection system

dea) Design basis

deb) Description of the characteristics of the control and safety protection system

dec) Materials of the control and safety protection system

ded) Inspections to be carried out in order to guarantee the characteristics of the control and safety protection system

dee) Evaluation of the control and safety protection system

def) Combined operation and evaluation of various reactivity control systems

e) Coolant system of the reactor and connected systems

ea) Description of the system

eb) Integrity of the coolant system of the reactor and connected systems

ec) Components of the coolant system

eca) Reactor pressure vessel and upper block

ecb) Main circulation pipeline

ecc) Main circulation pump

ecd) Pressurizer system

ece) Steam generator

ed) Connected systems

eda) Make-up water and boron control system

edb) Water treatment systems

edc) Emergency cooling systems

edd) Residual heat removal system

ede) Main steam and feedwater systems

edf) Organised leakage system

edg) Leakage monitoring system

edh) Continuous analytical measurement system

- edi)* Other systems
- ee)* Working and safety fittings and supporting structures
- f)* Safety protection systems and system components
 - fa)* Containment system
 - faa)* Design basis
 - fab)* Cooling and pressure reduction systems of the containment
 - fac)* Isolation system of the containment
 - fad)* Technical solutions for handling serious accidents
 - fae)* Containment leakage monitoring
 - faf)* Evaluation of containment
 - fb)* Emergency core cooling system
 - fba)* High-pressure emergency core cooling system
 - fbb)* Low-pressure emergency core cooling system
 - fbc)* Passive cooling systems
 - fc)* Systems providing for the presence of control room personnel
 - fd)* Emergency feedwater supply system
 - fe)* Other safety protection systems
- g)* Measurement, and control and instrumentation technology
 - ga)* Measurement, and control and instrumentation systems and system components included in safety classes and their functions
 - gb)* Emergency shutdown system of the reactor
 - gc)* Measurement, and control and instrumentation technology of safety protection systems and system components
 - gd)* Measurement, and control and instrumentation technology of systems providing for safe shutdown and maintaining the shutdown condition
 - ge)* Control rooms and their design, equipment providing information to the control rooms
 - gf)* Other measurement technology systems and system components associated with safety
 - gg)* Control and instrumentation systems not associated with safety
- h)* Electricity supply systems
 - ha)* Design basis for electricity supply required for the implementation of safety functions

- hb)* Off-site electrical supply system
- hc)* On-site electrical supply system
 - hca)* AC power supply
 - hcb)* DC power supply
 - hcc)* Main equipment types
 - hcd)* Cabling and cable paths
 - hce)* Grounding, overvoltage and lightning protection
- i)* Other systems and buildings
 - ia)* Storage and handling of fuel elements
 - iaa)* Storage of fresh fuel elements
 - iab)* Storage of spent fuel elements
 - iac)* Cooling and cleaning of the water of the spent fuel pool
 - iad)* Handling of fuel elements
 - ib)* Water systems
 - iba)* Cooling water systems required for the performance of the safety function
 - ibb)* Demineralised water-making and storage system
 - ibc)* Municipal and health care water supply systems
 - ibd)* Condensate storage system
 - ic)* Auxiliary process systems
 - ica)* Make-up water, boron supply and water operational systems
 - icb)* Sampling system
 - icc)* Steam generator surge system
 - icd)* System for receiving radioactive releases
 - ice)* Air and gas systems
 - id)* Diesel generator auxiliary systems
 - ida)* Fuel supply system
 - idb)* Cooling water supply system
 - idc)* Startup system
 - idd)* Lubricating oil supply system
 - ide)* Suction and exhaust system
 - ie)* Ventilation and air conditioning systems

- iea)* Ventilation systems of the controlled zone
- ieb)* Ventilation system of the spent fuel pool
- iec)* Ventilation systems for the management and storage of liquid and solid radioactive wastes
- ied)* Ventilation systems of safety protection systems and system components
- iee)* Ventilation system of the turbine building
- if)* Fire protection systems
- ig)* Communications systems
- ih)* Lighting systems
- ii)* Buildings associated with the facility
- ij)* Hoisting machinery
- j)* Supply, steam and energy conversion systems
 - ja)* Turbine and generator system
 - jb)* Fresh steam system
 - jc)* Main and auxiliary condensate system
 - jd)* Feedwater system
 - je)* Auxiliary steam system
 - jf)* Other systems
 - jfa)* Turbine condensers
 - jfb)* Condenser vacuum system
 - jfc)* Turbine stuffing box system
 - jfd)* Condenser cooling water system
- k)* Radioactive waste management
 - ka)* Determination of discharge sources
 - kb)* Liquid waste management systems
 - kba)* Design basis
 - kbb)* Description of the system
 - kbc)* Discharge values
 - kc)* Gaseous waste management systems
 - kca)* Design basis
 - kcb)* Description of the system

kcc) Release values

kd) Solid waste management systems

kda) Design basis

kdb) Description of the system

ke) Concept for the management of wastes produced during the management and elimination of accidents, severe accidents and very severe accidents

kea) Estimation of the quality and quantity of wastes produced

keb) Description of a comprehensive waste management concept

kf) System for monitoring and sampling the radioactivity of process systems and environmental releases

kfa) Design basis

kfb) Description of the system

kfc) System for monitoring and sampling environmental releases

kfd) System for monitoring and sampling the radioactivity of process systems

kg) Evaluation

l) Radiation protection

la) Ensuring compliance with the lowest reasonably achievable level

laa) Commitment of the management

lab) Design considerations

lac) Operational considerations

lb) Radioactive radiation sources

lba) Solid and liquid radioactive materials

lbb) Radioactive materials in the form of aerosols and noble gases

lbc) Technological description of radiation hazardous activity(ies)

lc) Design requirements for radiation protection

lca) Design requirements

lcb) Design of shielding

lcc) Ventilation

lcd) Installed radiation and aerosol monitoring system

lce) Optimization aspects applied during the design of radiation protection

lcf) System of requirements for the determination of supervised and controlled areas and planned determination of the supervised and controlled areas

ld) Dose calculations

lda) Determination of on-site dose exposures

ldb) Determination of off-site dose exposures

ldc) Dose estimations for representative groups:

ldca) determination of doses from external radiation, in the given case by specifying the type of the radiation,

ldcb) determination of activity concentration of the released radionuclides in foodstuffs and drinking water, or in other relevant environmental element

le) Radiation protection programme

lea) Organisation implementing the programme

leb) Installed radiation protection monitoring system

lec) Sampling radiation protection monitoring system

led) Radiation protection procedures and methods

lf) Evaluation

m) Management of operation

ma) Organogram

maa) Management

mab) Organisation verifying compliance with the safety requirements

mac) Operating organisation

mad) Technical support organisation

mae) Personnel requirements and the way of meeting them

mb) Training and personnel training programme

mc) Preliminary emergency preparation plan

md) Reviews and audits

mda) Review performed by the designated organisation of the Licensee

mdb) Review performed by an external organisation independent of the Licensee

mdc) Review and audit programmes

me) Procedures

mea) Administrative instructions

meb) Technical instructions

mf) Preliminary physical protection plan

n) Commissioning programme

- na)* Preliminary scope of the commissioning programme
- nb)* Testing and operating experience to be used for the development of the commissioning programme
- nc)* Preliminary plan for the provision of personnel required for commissioning
- nd)* Orders of the authorities taken as a basis
- ne)* Preliminary schedule of the commissioning programme
- nf)* Preliminary plan for the review of operating, breakdown and emergency instructions
- o)* Safety analyses
 - oa)* Normal operating condition (DBC1)
 - ob)* Anticipated operational occurrences (DBC2) and design basis accidents (DBC3 and 4)
 - oba)* Initiating events
 - obb)* Verification of the input data and computer programs used for analyses and of validity, modelling considerations, initial and boundary conditions, and acceptance criteria
 - obc)* Analysis results
 - oc)* Complex accidents (DEC1)
 - oca)* Initiating events and their categorisation
 - ocb)* Verification of the input data and computer programs used for analyses and of validity, modelling considerations, initial and boundary conditions, and acceptance criteria
 - occ)* Analysis results
 - od)* Severe accidents (DEC2)
 - oda)* Initiating events and their categorisation
 - odb)* Verification of the input data and computer programs used for analyses and of validity, modelling considerations, initial and boundary conditions, and acceptance criteria
 - odc)* Analysis results
 - oe)* Substantiation of the application of deterministic and probabilistic safety analyses
- p)* Preliminary Operational Limits and Conditions and their substantiation
- q)* Quality assurance
 - qa)* Checking of designs and documentation

- qb)* Qualification of contractors and audits
- qc)* Checking of materials, equipment, instruments and services
- qd)* Process control
- qda)* Inspection of construction processes
- qdb)* Production control
- qdc)* Inspection of installation
- qdd)* Inspection of the inactive commissioning activities of systems
- qde)* Review of facility-level comprehensive commissioning tests
- qef)* Testing and material testing
- qf)* Verification of control and instrumentation equipment and software
- qg)* Inspection of transport, handling and warehousing
- qh)* Verification of non-compliances and corrective actions
- qi)* Quality assurance documentation
- r)* Human-machine relations
- s)* Preliminary plan for the decommissioning of the nuclear power plant and its units
- sa)* Decommissioning concept plan
- sb)* Radiation sources
- sc)* Radiation monitoring during decommissioning
- sd)* Recoverable materials
- se)* Systems, equipment and organisation of decommissioning.

1.2.3.0300.

1.2.3.0310. Further recommendations for the structure and contents of the Preliminary Safety Analysis Report are set forth in the guidelines.

1.2.3.0400. The effective local construction regulations and regulation plan related to the site shall be attached to the license application.

1.2.3.0410. The effect of the construction activities associated with the period of operation on the nuclear safety of the operational facility shall be described and evaluated in the Preliminary Safety Analysis Report of the facility for the interim storage of spent fuel. The scope and contents of the modification activities associated with extension shall be described in the Preliminary Safety Analysis Report.

1.2.3.0500. The schedule and accomplishment of construction activities shall be demonstrated in the license application in such detail that based on the

aforementioned the nuclear safety authority is able to determine retention points applicable for monitoring purposes, and to plan its monitoring.

1.2.3.0600. In the case of a facility for the interim storage of spent fuel, the Preliminary Safety Analysis Report shall be completed for the planned complete structure of the facility. If it becomes necessary to modify the Preliminary Safety Analysis Report during operation for the extension of the facility, the contents of the new version of the Preliminary Safety Analysis Report may be narrowed down to the scope of the further construction activities required for completeness, taking into account the operation license and the underlying Final Safety Analysis Report.

1.2.4. Commissioning license

Scope of license

1.2.4.0100. The commissioning license shall authorise to perform the following activities:

a) the first placement of fuel assemblies into the nuclear reactor, in the case of interim spent fuel storage facility the first placement of spent fuel into the storage positions of the interim storage facility,

b) scheduled execution of commissioning programmes which verify the operation of the nuclear facility according to the design, and the performance of tests of systems and system components important to nuclear safety under active circumstances, and

c) operation at rated power in the case of nuclear power plant units and research reactors or operation with the stored spent fuel in the case of the interim spent fuel storage facility, from the date of the successful execution of the commissioning programme to the date specified in the license.

1.2.4.0200. The commissioning license shall lose its effect once the operation license of the nuclear facility becomes final.

1.2.4.0210. In the case of the extension of the facility for the interim storage of spent fuel, the commissioning license shall authorise its holder to put into operation the module awaiting commissioning; the operation of the new module shall require the obtaining of an operation license specified in Section 1.2.5.

1.2.4.0300. The commissioning license shall be effective for 12 months from its date of issuance; however, the nuclear safety authority may determine its duration for a period other than 12 months based on a justification specially formulated in its decision, taking into consideration the time necessary for the preparation and performance of commissioning and the specifics of the operation of the facility.

Content requirements of the license application

1.2.4.0400. The following shall be demonstrated in the license application:

- a) the construction of the nuclear facility is in compliance with the design,
- b) the realized conditions are in compliance with the legal requirements,
- c) the modifications accomplished as compared with the content of the Preliminary Safety Analysis Report are well-founded and licensed if the modifications are subject to licensing,
- d) the elimination of nuclear safety related deficiencies identified during construction,
- e) the planned commissioning activity is suitable to demonstrate that the operation of the nuclear facility is in compliance with the design and nuclear safety requirements, in the case of the extension of the facility for the interim storage of spent fuel, taking into account the available facility-level complex commissioning results,
- f) the conditions for safe operation are ensured, i.e. the nuclear facility is suitable for commissioning after the successful completion of the planned commissioning activities,
- g) the safe interim storage or final disposal of radioactive waste generated in the nuclear facility – including the spent fuel in the case of a nuclear reactor – in accordance with the most recent proven scientific results, international expectations and experience is provided,
- h) in the case of the extension of a facility for the interim storage of spent fuel, the connected systems have been modified and the modification conforms to the relevant regulations,
- i) in the case of a facility for the interim storage of spent fuel, carrying out the planned commissioning activities does not reduce the nuclear safety of the operating facility, and the conditions of the safe implementation of commissioning are fulfilled,
- j) the instruments suitable to measure the parameters necessary to assess the radioactive contamination of the environment and the radiation exposure of the population, and the respective procedures are available,
- k) the radiation protection system ensures that the radiation exposure of workers and the population is not higher than justifiable,
- l) the installed radiation protection measurement system appropriately indicates the amount of radioactive isotopes possibly released to the environment, and
- m) the applied optimization methods are in compliance with the ALARA principle.

1.2.4.0500. The following shall be attached to the license application with the exception of the case of the extension of a facility for the interim storage of spent fuel:

- a) the preliminary version of the Final Safety Analysis Report,
- b) the Operational Limits and Conditions document,
- c) the document describing the procedures which provide for the maintenance of the condition of systems, structures and components important to nuclear safety as specified in the design and the Final Safety Analysis Report,
- d) the emergency operating procedures,
- e) the documents of accident management procedures,
- f) the Nuclear Emergency Preparedness and Response Plan of the nuclear facility,
- g) the detailed commissioning programme of the nuclear facility,
- h) the safety policy according to Section 8(1),
- i) the Workplace Radiation Protection Rules.

1.2.4.0510. In the case of a facility for the interim storage of spent fuel, the document specified in Section 1.2.4.0500 f) shall be approved in the modification licensing procedure under Section 1.4 after the commissioning license of the first module. The existence of the approved document shall be a precondition for granting the commissioning license.

1.2.4.0520. In the case of the extension of a facility for the interim storage of spent fuel, the following shall be attached to the license application:

- a) preliminary version of the sections of the Final Safety Analysis Report affected by the extension,
- b) preliminary version of the document entitled 'Operating Conditions and Limits' affected by the extension,
- c) operating instructions regulating the elimination of accidents, and
- d) commissioning programmes and a preliminary schedule for the implementation of the programmes.

1.2.4.0600. The changes as compared to the specifications set out in the preliminary version of the Preliminary Safety Analysis Report shall be summarised in a separate chapter in the preliminary version of the Final Safety Analysis Report.

1.2.5. Operation license

Scope of license

1.2.5.0100. In the possession of operation license, the nuclear facility can be operated under the conditions and for the duration specified in the license.

1.2.5.0200. The nuclear safety authority shall set the duration of the operational license taking into consideration the operational features of the relevant nuclear facility and other circumstances, but it shall not extend beyond the designed service life of the nuclear facility. Operation beyond the designed service life requires a new license according to clause 1.2.6.

1.2.5.0300. The operation license shall lose effect, if

a) the conditions specified in the license are not fulfilled;

b) the nuclear reactor is in a cooled down, subcritical condition continuously for a period of more than 12 months in the nuclear facility within which the reactor is operated;

c) the energy generating part of the nuclear power plant unit is depressurized for more than 12 months; furthermore

d) the interim spent fuel storage facility is empty, without spent fuel continuously for more than 12 months.

1.2.5.0400. In the case of the expiration of the operation license or termination of its duration, it shall be necessary to obtain a new operation license as specified in Section 1.2.7 to further operate the nuclear facility.

1.2.5.0500. In the case of Category 1 modification according to Section 4.8 of Annex 4, Section 5.3.13 of Annex 5 and Section 6.3.9 of Annex 6, the modification of the operation license of the nuclear facility shall be requested from the nuclear safety authority. In addition to the fulfilment of the content requirements with regard to the new nuclear facility, the modifications necessary to the documents substantiating the original operation license shall be presented in the license application, and the Modification Assessment Report shall be submitted according to Section 1.4.1.1800.

Content requirements of the license application

1.2.5.0600. In the license application:

a) the modifications concerning the Final Safety Analysis Report which became necessary during the execution of the commissioning programme shall be summarised and substantiated,

b) the availability of data related to "0" conditions necessary to monitor and assess the change in conditions of systems, structures and components important to nuclear safety shall be demonstrated,

c) it shall be demonstrated that the safe interim storage or final disposal of radioactive waste generated by the nuclear facility – including spent fuel in the

case of nuclear reactors – is in accordance with the latest proven scientific results and international expectations and experience,

d) it shall be demonstrated how the licensee ensures the necessary resources to maintain nuclear safety during the requested validity of the operation license,

e) in the case of a facility for the interim storage of spent fuel, the safety of the operational facility shall be demonstrated by taking into account the implementation and commissioning activities associated with the operational phase of the life cycle of the nuclear facility, and

f) it shall be verified that each type of radiation hazardous work is adequately regulated in the Workplace Radiation Protection Rules, in compliance with the ALARA principle.

1.2.5.0700. The following shall be attached to the license application, as having been reviewed based on the experience of the activities performed on the basis of the commissioning license:

a) the updated Final Safety Analysis Report which shall verify – taking into consideration the results of commissioning tests – that

aa) the nuclear facility operates in compliance with the valid design basis,

ab) the inspection, management, emergency operating and accident management provisions necessary for the safe operation are suitable for the attainment of the formulated objectives, and

ac) the safe operation is ensured under the operational limits and conditions set out in the Final Safety Analysis Report,

b) Operational Limits and Conditions document,

c) the document describing the procedures which ensure the maintenance of the condition of systems and system components important to nuclear safety as specified in the designs and the Final Safety Analysis Report,

d) emergency operating procedures,

e) accident management procedures,

f) Nuclear Emergency Preparedness and Response Plan of the nuclear facility, and

g) the Workplace Radiation Protection Rules.

1.2.5.0800. In the case of a facility for the interim storage of spent fuel, the emergency operating instructions and, if warranted, the accident management guidelines shall also be extended to the sphere of construction and commissioning activities associated with the extension.

1.2.5.0900. The dose constraint value shall be revised at the issuance of the operation license, but at least in the frame of the PSR.

1.2.6. Licensing of operation beyond the designed service life

General requirements

1.2.6.0100. Except for nuclear power plants, operation of nuclear facilities beyond the designed service life shall be licensed by issuance of a new operation license based on the last periodic safety review before the expiration of the designed service life then based on all subsequent periodic safety reviews. The new license shall be effective until the next periodic safety review of the nuclear facility.

1.2.6.0200. In the case of nuclear power plant units licensing of operation beyond the designed service life shall take place by the issuance of a new operation license.

1.2.6.0300. In the case of nuclear power plant units, the following items belong to the scope of licensing of operation beyond the designed service life:

- a) systems, structures and components with safety function,
- b) those non safety class components, the failure of which can prevent a system or component in fulfilling its safety functions, and
- c) systems, structures and components taken into the scope with ad hoc authority decision.

1.2.6.0400. Prior to licensing, the licensee shall prepare and implement a programme to create the conditions for the operation of the nuclear facility beyond the designed service life and to demonstrate its operability.

1.2.6.0500. The licensing of operation of the nuclear power plant unit beyond the designed service life is based on the following principles:

a) during the preparation for the operation licensing of the nuclear power plant unit beyond the designed service life and during the operation beyond the service life, the safe operability shall be continuously maintained in compliance with the legal and regulatory provisions, i.e. the current issues arising with regard to the operation of the nuclear facility shall be resolved within the framework of the effective operation license of the nuclear facility;

b) during the operation of nuclear power plant unit beyond the designed service life, the necessary safety margins of systems, structures and components taken into consideration in the safety analyses shall not be used with reference to the forthcoming licensed operation time termination date;

c) the licensee shall commence within the designed service life and perform continuously onwards those activities set out in Section 4.6 of Annex 4 intended

to maintain the technical conditions, furthermore shall systematically monitor and assess the efficiency of that activities;

d) taking point c) into consideration the demonstration of operability beyond the designed service life is essentially limited to the demonstration of the suitability of passive and long-lived system components;

e) the safety improving measures, which can be derived from modern international requirements, shall be determined within the framework of the periodic safety review, according to the applicable regulations.

Programme for the implementation of operability conditions of nuclear power plant units beyond the designed service life

1.2.6.0600. The programme meant for the establishment of nuclear power plant unit operability beyond the designed service life (hereinafter: Service Life Extension Programme – SLE Programme) may be submitted simultaneously with regard to one or more units of the nuclear power plant. A minimum of 20 years operating experience shall be analysed in the SLE Programme. In the case of an SLE Programme to be submitted for several nuclear power plant units, the operation service life of the first commissioned nuclear power plant unit shall be taken into consideration.

1.2.6.0700. The SLE Programme shall contain the scheduled period of extended service life.

1.2.6.0800. The content of the SLE Programme shall be determined according to the content requirements of the new license application of the operation beyond the scheduled service life. It shall be proven in the programme that by its execution, the criteria being the basis for the effective operation license are fulfilled for the entire extended service life in accordance with the requirements of the Nuclear Safety Code. The requirements already fulfilled at the time of submission shall be outlined in the SLE Programme. The information establishing the fulfilment of the requirements shall be presented or referred to. Additionally, the fulfilment of further requirements and those activities – in addition to their scheduled performance – which are intended to be performed in order for the full implementation of the SLE Programme shall be outlined.

1.2.6.0900. The licenses required for the performance of modifications arising during the execution of the SLE Programme shall be obtained separately during the existence of the effective operation license, in accordance with the applicable regulations for the licensing of the relevant activity.

Operation of nuclear power plant units beyond the designed service life

1.2.6.1000. The operation beyond the designed service life shall be licensed under a new operation license issued at the request of the licensee. The license

application shall be submitted per nuclear power plant units at least one year prior to the expiration of the valid operation license.

Scope of operation license

1.2.6.1100. In possession of the new operation license the nuclear power plant units may be operated under the conditions and for the duration as specified in the license.

1.2.6.1200. The nuclear safety authority shall set the duration of the license taking into consideration nuclear safety and other circumstances of operation, but it cannot exceed the scheduled duration demonstrated in the documentation substantiating the operation beyond designed service life.

1.2.6.1300. Should the operation license become invalid within the period licensed for operation beyond the designed service life by the nuclear safety authority, then the nuclear power plant unit shall only be operated again once a new operation license is obtained in the procedure set out in Section 1.2.7.

Content requirements of license applications

1.2.6.1400. The following shall be presented in the license application:

- a)* general information related to the nuclear facility and the operator thereof,
- b)* definition of systems, structures and components belonging to the scope of licensing the operation beyond the designed service life,
- c)* comprehensive review on ageing management of passive and long-lived components,
- d)* management of time limited ageing analyses,
- e)* required modifications of the Final Safety Analysis Report,
- f)* required modifications of the Operational Limits and Conditions document,
- g)* modifications of further documents upon which the operation license is based,
- h)* demonstration that the SLE Programme has been executed and based on those activities specified in it and throughout the service life
 - ha)* the nuclear power plant unit is in a safely operable condition, and that the technical and administrative conditions necessary for its long-term maintenance are provided for, and
 - hb)* the licensee possesses the resources necessary for the long-term maintenance of nuclear safety, furthermore
 - i)* the planned duration of operation beyond the designed service life as substantiated by analyses.

1.2.6.1500. The actualized version of the following documents shall be attached to the license application:

- a) Operational Limits and Conditions document,
- b) the document describing the procedures which ensure that the condition of systems, structures and components important to nuclear safety as specified in Section 4.6 of Annex 4, are maintained,
- c) emergency operating procedures ,
- d) accident management procedures,
- e) Nuclear Emergency Preparedness and Response Plan of the nuclear facility, and
- f) the Workplace Radiation Protection Rules.

1.2.6.1600. The recommendations regarding the content of documentation substantiating the operation beyond the design service life and content of the SLE Programme shall be included in a guideline.

1.2.7. Regaining withdrawn or invalid operation licenses

1.2.7.0100. In the event of the termination of the effect of the operation license as set out in Section 1.2.5 or 1.2.6, a new license application shall be submitted. If the licensee submits the application on time as set in Section 1.2.6.0900, and the procedure regarding the licensing of operation beyond the design service life is still in progress at the expiration of the duration of the previous operation license, no new operation license application is necessary to be submitted. In such cases, the licensing procedure as set in Section 1.2.6 shall be terminated.

Scope of license

1.2.7.0200. Once in the possession of a final operation license, the nuclear facility may be operated under the conditions and for the duration as specified in the license, in the case of the existence of other licenses specified by legislation.

1.2.7.0300. The nuclear safety authority shall lay down the duration of the license while taking into consideration nuclear safety and other circumstances of operation, but it shall not be longer than the remaining period of operation as specified in the license regarding the designed or beyond designed operation of the nuclear facility operated whilst in the possession of an operation license.

Content requirements of the license application in the case of the termination of the effect of the license

1.2.7.0400. In the license application, it shall be verified that

- a) the nuclear facility is in a safely operable condition, and that the technical and administrative conditions necessary to maintain this in the long term are ensured and the radiation protection rules are appropriate,

b) the licensee possesses the necessary resources to maintain long-term nuclear safety, and

c) the causes and circumstances resulting in the termination or limitation of the scope of the license are eliminated, furthermore that the provisions of the nuclear safety authority are fulfilled.

1.2.7.0500. The following shall be attached to the license application:

a) the required modifications to the documents substantiating the original operation license specified in Section 1.2.5 in the period within the designed service life, and in Section 1.2.6 in the period beyond the designed service life, justification for the modifications, and the updated version of documents substantiating the original operation license, and

b) documents required to demonstrate the compliance with Section 1.2.7.0400 and required by the nuclear safety authority.

1.2.8. Termination license of nuclear facilities

General provisions

1.2.8.0100. The licensee shall submit the Preliminary Decommissioning Plan for the nuclear facility following each update to the nuclear safety authority for information during the period of operation.

1.2.8.0200. The termination documentation of nuclear facilities located on adjoining sites, being in technological or organisational connection, may be submitted to the nuclear safety authority separately, however, the interactions concerning nuclear safety shall be demonstrated therein.

1.2.8.0300. Should the licensee make a decision concerning the final shutdown of the nuclear facility, it shall be announced immediately to the nuclear safety authority. Within this announcement the licensee shall set out the schedule for the termination of the nuclear facility.

1.2.8.0400. The licensee shall submit the Final Decommissioning Plan specified in Section 8.2.4 of Annex 8 to the nuclear safety authority for information within two years following the final shutdown of the nuclear facility.

1.2.8.0500. In the implementation period of decommissioning, the licensee shall submit the Final Decommissioning Plan which forms the basis for the activities, to the nuclear safety authority for information following each update.

1.2.8.0600. The nuclear safety authority oversight of the termination of nuclear facilities covers the issuance of the final shutdown license and decommissioning license at request, and the necessary approval for the commencement of certain activities specified therein if the respective specific conditions exist, issuance of the resolution on termination of nuclear safety oversight according to Section 1.2.9., and inspection of compliance with the licenses and the resolution.

Final shutdown license

1.2.8.0700. The final shutdown license entitles the holder of the operation license to perform activities related to the termination of operating activity – including the termination of storage in the interim spent fuel storage facility – and activities necessary for the preparation of decommissioning, performed as part of the preparation.

1.2.8.0800. In the final shutdown license the nuclear safety authority shall

a) list the further nuclear safety licenses according to the legislation required during the final shutdown and the preparation for decommissioning within the framework of operation, based on the activities indicated in the license application, as scheduled for the given period,

b) provide for those activities the commencement of which are bound to obtaining a separate nuclear safety authority approval,

c) specify the conditions of approval, and

d) determine the regular and ad hoc reports and the content thereof to be submitted in the course of the preparation for final shutdown and decommissioning.

1.2.8.0900. The nuclear safety authority shall issue the license for the time period specified in the final shutdown and plan for preparation for decommissioning within the frame of the operation of the nuclear facility, but for a maximum of 10 years. If the preparation phase of decommissioning also includes a guarded supervision period, the duration of the license may be extended once or more on request, to a maximum of 10 years per occasion, with a maximum to the total guarded supervision period.

Content requirements of the license application

1.2.8.1000. The license application shall contain:

a) an introduction as to how the termination of operation activity fits into the potentially existing national and industrial termination strategy and that it is in conformity with the termination strategy of the nuclear facility and complies with the nuclear safety regulatory provisions,

b) the final shutdown and decommissioning preparation plan for the nuclear facility,

c) condition maintenance and control tasks plan to the necessary extent and the foundation thereof during the potential guarded supervision period between the final shutdown and the decommissioning of the nuclear facility, and

d) radiation protection measures and the foundation thereof to be achieved during the final shutdown of the nuclear facility.

1.2.8.1100. The following shall be attached to the license application

- a) updated version of the Final Safety Analysis Report,
- b) the updated Nuclear Emergency Preparedness and Response Plan of the nuclear facility,
- c) dependent upon the characteristics of the nuclear facility the symptom-based emergency operating procedures and the actual version of the accident management procedures,
- d) in the case of nuclear power plant units, the license of the public administration body responsible for the official supervision of electricity production with regard to the termination of electrical power generation,
- e) the radiation protection measures to be implemented during the final shutdown of the nuclear facility and their substantiation, and
- f) the Workplace Radiation Protection Rules,
- g) the plan of the final shutdown of the nuclear facility and the updated version of the decommissioning plan,
- h) in the possible guarded supervision period between the final shutdown and decommissioning of the nuclear facility the plan of actions meant to maintain the condition and its substantiation.

Decommissioning license

1.2.8.1200. In the event of the existence of other licenses specified by legislation and the fulfilment of the conditions laid down in the license, the decommissioning license authorises for the purpose of termination of the nuclear facility, the termination and dismantling of the systems, structures and components thereof, the demolition of buildings and building structures taking into consideration Section 1.5.1.0400, furthermore the performance of other activities necessary for the limited or unlimited utilisation of the site. The final criteria for decommissioning activities are contained within the license.

1.2.8.1300. Under the decommissioning license the nuclear safety authority shall

- a) list the further nuclear safety licenses required by legislation during decommissioning taking into consideration the activities to be performed during decommissioning as indicated in the license application,
- b) address those activities the commencement of which are bound to obtaining a separate nuclear safety regulatory approval,
- c) determine the conditions of approval, and
- d) determine the regular and ad hoc reports to be submitted during decommissioning and the content thereof.

1.2.8.1400. The duration of the decommissioning license is 10 years. If the decommissioning phase includes a guarded supervision period, then the license duration may be longer maximum by the length of guarded supervision period.

Content requirements of the license application

1.2.8.1500. The license application shall contain:

- a) the plan of decommissioning activity,
- b) compliance of the Final Decommissioning Plan with the termination strategy, including the specification of the planned final condition,
- c) technical, organisational and other conditions necessary to implement the plan, furthermore the methods how these are provided,
- d) method of preserving experience and knowledge,
- e) radiation protection objectives and how they are met, and
- f) method of storage, disposal, processing, release and transportation of radioactive waste.

1.2.8.1600. The following shall be attached to the license application:

- a) the Final Decommissioning Plan and its substantiation,
- b) the Decommissioning Safety Analysis Report as specified in Annex 8 and the Nuclear Emergency Preparedness and Response Plan related to the decommissioning period, and
- c) the Workplace Radiation Protection Rules.

1.2.9. Termination of the nuclear safety authority oversight of nuclear facilities

1.2.9.0100. The Final Decommissioning Safety Report specified in Annex 8 shall be submitted to the nuclear safety authority by the licensee.

1.2.9.0200. Based on the Final Decommissioning Safety Report, the licensee may request that the nuclear safety authority terminate the nuclear safety authority oversight of the nuclear facility or that of the nuclear facility and site jointly. The nuclear safety authority oversight of the site may be terminated, if

- a) the final state set out in the termination strategy of all nuclear facilities located on the site is achieved, and
- b) neither the licensee nor any other party wishes to establish a new nuclear facility on the site.

1.2.9.0300. The nuclear safety oversight of the nuclear facility and its site may be terminated by a release licensed for unlimited usage, or by a release licensed for usage with limitations. The criteria of release licensed for unlimited usage and usage with limitations are specified in law.

1.2.9.0400. If the licensee requests the release of the nuclear facility and its site to be licensed for usage with limitations, then the licensee shall develop and submit its proposal related to the limitations, furthermore the draft of institutional monitoring system ensuring the observance thereof. The purpose, requirements and method of supporting of the institutional monitoring system are included in Annex 8.

1.2.9.0500. The decision containing the regulatory decision on the release licensed for usage with limitations shall contain the limitations, and obligations related to the observance thereof furthermore related to the construction and operation of the institutional monitoring system.

1.2.9.0600. In the decision which terminates the nuclear safety oversight of the nuclear facility and the site with a release licensed for unlimited or limited usage, the content of documentation to be obligatory retained, the required number of copies, time and location of preservation of the document types shall be specified. The nuclear safety authority oversight, the liability and status of the licensee shall terminate only after archiving the documentation of the nuclear facility according to the decision.

1.2.9.0700. After release licensed for limited usage, the site and the nuclear facility oversight, within this the monitoring of the observance of limitations and the operation of the institutional monitoring system shall be performed by the administrative body specified by legislation.

1.2.10. The replacement of the licensee of a nuclear facility

1.2.10.0100. The announcement of the intention to replace the license holder shall be considered a new license application, and a new licensing procedure shall be conducted. In the cases regulated in Section 7(1) of the Atomic Act, the original copy of the government's preliminary consent in principal shall be attached to the application. In the case of documents named as part of the application in the regulation but not concerned by the change in the person of the licensee, it is enough to refer – ensuring retrieval – to the previous procedure within the frame of which the documentation was submitted to the nuclear safety authority. The declaration of the current licensee stating that the current licensee agrees with the person of the new licensee and that after issuing the new license, the current licensee does not wish to remain license holder shall be attached to the application.

1.2.10.0200. In the procedure, the nuclear safety authority shall examine the existence of technical, technological, material and human resources necessary for the safe performance of at least the activity specified in the license.

1.2.11. Restart permit of a nuclear power plant unit following refuelling

1.2.11.0100. Following refuelling, the nuclear reactor may only be made critical once the nuclear safety authority permit is obtained for that.

Scope of the permit

1.2.11.0200. The permit shall provide authorisation to bring the nuclear power plant unit to a hot zero power state, and to later load it to the rated capacity. Following the achievement of rated capacity, the permit shall automatically become invalid and the operation shall be performed according to the nuclear power plant unit operation license.

Content requirements of the permit application

1.2.11.0300. In the permit application it shall be demonstrated that following refuelling, the nuclear power plant unit complies with the nuclear safety requirements, the safety analyses and the Operational Limits and Conditions document remain valid for the new core, and that the maintenance, repair and modification works and inspections necessary for the safe operation of the nuclear power plant unit have been performed.

1.2.11.0400. The following shall be attached to the permit application:

- a) maintenance schedule, and
- b) the plan and preliminary cartograms of fuel assembly movement, the preliminary load map and safety substantiation of the active core.

1.2.11.0500. The permit application for restarting and the documents according to Section 1.2.11.0400 shall be submitted at least two weeks before the scheduled outage of the nuclear power plant unit for refuelling.

1.2.11.0600. The following shall be submitted to the nuclear safety authority for its information at least one week prior to the scheduled date for rendering the active core critical, with the content in compliance with the condition of the second day prior to submission:

- a) the load map, reactor physics characteristics, startup calculations and safety foundation of new active core,
- b) the cartograms of the arrangement of fuel assemblies,
- c) list of modifications implemented during maintenance or under implementation,
- d) the executed technical safety reviews, in-service inspection, corrosion inspection, and the results thereof, furthermore the list and schedule of any remaining inspections,
- e) the executed unit startup tests and the results thereof, furthermore the list and schedule of any remaining tests,

f) the summary of completed scheduled maintenance and repair works, the works over schedule and the necessity thereof and a brief justification of the cancelled works, and

g) summary of nuclear safety related events which occurred during maintenance and the management thereof.

1.3. NUCLEAR SAFETY LICENSES RELATED TO NUCLEAR SYSTEMS, STRUCTURES AND COMPONENTS

1.3.1. General provisions

1.3.1.0100. The provisions of Section 1.3 shall be applied during the construction phase of the nuclear facility's lifecycle.

1.3.1.0200. In the case of construction of a nuclear facility with a modular system, the system components to be manufactured for the modules shall be licensed according to Section 1.3, while the buildings and building structures shall be licensed in accordance with the provisions of Section 1.5. At the modification of those existing systems, structures and components which have to serve the already operating and newly installed modules, the provisions of Section 1.4 shall be applied.

1.3.1.0300. The nuclear safety authority shall issue manufacturing or purchase type licenses for the manufacturing or procurement of system components if more than one are to be built in or installed which comply with the unified set of requirements derived from the location of installation, safety and seismic safety classification furthermore from the functions to be performed. With regard to the content and scope of the type license application the provisions of Sections 1.3.2 and 1.3.3 shall be applicable.

1.3.2. Manufacturing license

1.3.2.0100. The following may be manufactured based on a license issued by the nuclear safety authority:

- a) system components categorised into safety class 1,
- b) system components or pressure retaining equipment and pipelines categorised into safety class 2, provided that they are not procurable as a commercial product,
- c) software categorised into safety class 2, and
- d) pressure retaining equipment and pipelines categorised into safety class 3, subject to nuclear safety authority licensing obligation, provided that they are not procurable as a commercial product,
- e) safety class 3 components meant to manage DEC1-2 plant states,

f) in the case of a facility for the interim storage of spent fuel, system components and software belonging to the highly important and important safety classes, which cannot not procured as commercial products.

Scope of license

1.3.2.0200. The manufacturing license shall provide authorisation to manufacture the system component and deliver it to the site in the event that the conditions set out therein are fulfilled.

1.3.2.0300. The license shall have a specific duration which shall be determined by the nuclear safety authority taking into consideration the uniqueness of the manufacturing activity, the safety importance of the system component, and the time necessary for the preparation and performance of manufacturing, but its duration shall not exceed 5 years.

1.3.2.0400. The license may be extended once for a maximum period being the same as the duration of the license provided that the legal provisions remain unchanged.

Content requirements of the license application

1.3.2.0500. The license application shall contain:

a) the name, safety and seismic safety classification of the system component, and the name of the safety function performed by the system component;

b) the name of system to which the system component belongs, the location of installation, and the function of the system component in each designed operating condition of the nuclear power plant unit or the facility for the interim storage of spent fuel;

c) technical and quality requirements specified in legislation and the Preliminary Safety Analysis Report of the nuclear facility with regard to the system component, location of installation, safety and seismic safety class, to be taken into consideration during manufacturing;

d) designer statements testifying the compliance with nuclear safety requirements;

e) design basis, design specification of the system components and the respective substantiating documents;

f) design information;

g) technical description which summarises the technical solution in line with the nuclear safety requirements, the operation and the most important technical features;

h) the description of the manufacturing process and technology, in the case of software the software development process, the method and instrument description thereof;

i) the audit plan for manufacturing and software development, the name of the auditing body, and the documents certifying the competence of this body to undertake their controlling activity;

j) documents certifying the qualification of the designer and manufacturer;

k) information with regard to the product reference;

l) content requirements for the delivery documentation certifying the compliance of the system component;

m) provisions developed by the manufacturer or designer for the component, with regard to the Operational Limits and Conditions, furthermore provisions, criteria, methods, programmes and their schedules related to its operation and maintenance of its technical condition;

n) list of wearing and strategic spare parts necessary for the maintenance and repair of the system component; and

o) the reference number of the license issued by the nuclear safety authority in connection with the procedure, the name and identification numbers of documentation previously submitted by the licensee and used for the substantiation of the application.

1.3.3. Procurement license

1.3.3.0100. The following can be procured based on the license issued by the nuclear safety authority:

a) system components categorised into safety class 1 appearing as exceptions according to Sections 3.3.1.1800 and 3a.3.1.2200;

b) system components, pressure retaining equipment and pipelines categorised into safety class 2;

c) pressure retaining equipment and pipelines categorised into safety class 3, subject to nuclear safety authority licensing obligation; and

d) in the case of a facility for the interim storage of spent fuel, system components belonging to the highly important and important safety classes.

Scope of license

1.3.3.0200. The procurement license shall provide authorisation to procure system components as a commercial product and deliver it to the site in the event that the conditions set out therein are fulfilled.

1.3.3.0300. The license shall have a specific duration, which shall be determined by the nuclear safety authority taking into consideration the safety importance of the system components to be procured, the uniqueness of the construction and manufacturing thereof, the schedule of procurement, and the time necessary for the preparation and performance of the procurement, but it shall not exceed 5 years.

1.3.3.0400. The license may be extended once, for a maximum period being the same as the duration of the license provided that the legal provisions remain unchanged.

Content requirements of the license application

1.3.3.0500. The license application shall contain:

a) the name, safety and seismic safety classification of the commercial product, and the name of the safety function performed by the system component;

b) the name of the system into which the commercial product is intended to be installed, the location of installation, and the task to be performed in each designed operation state of the nuclear power plant unit and the facility for the interim storage of spent fuel;

c) technical and quality requirements specified in legislation and the Preliminary Safety Analysis Report of the nuclear facility with regard to the planned location of installation of the commercial product, safety and seismic safety class, to be taken into consideration during procurement;

d) design basis, design specification of the system component, and the respective substantiating documents;

e) design information;

f) technical description which summarises the technical solution in line with the nuclear safety requirements, the operation and the most important technical features;

g) documents certifying compliance with the requirements of point *c)*, the instruments of which may be the following:

ga) analysis and assessment of compliance with the requirements,

gb) analysis and assessment of data of application references,

gc) documents of type test performed by the manufacturer or independent quality certification laboratory,

gd) the statement of the commercial product designer, or

ge) design and manufacturing documentation of commercial products;

h) the inspection plan for the final acceptance of the commercial product, name of the inspection body and the documents certifying the compliance of the inspection body to perform the activity;

i) documents certifying the qualification of the manufacturer;

j) information regarding the application reference of the commercial product;

k) provisions developed by the manufacturer or designer for the component with regard to operational limits and conditions of the commercial products, furthermore provisions, criteria, methods, programmes and their schedules related to its operation and maintenance of its technical condition;

l) the list of wearing and strategic spare parts necessary for the maintenance and repair of the system component;

m) the content requirements of the handover documentation certifying the compliance of the commercial product; and

n) the reference number of the license issued by the nuclear safety authority in connection with the procedure, the name and identification numbers of documentation previously submitted by the licensee and used for the substantiation of the application.

1.3.4. Assembly license

1.3.4.0100. The assembly of

a) system components included in Safety Class 1 and

b) in the case of the extension of a facility for the interim storage of spent fuel, system components provided with a manufacturing and procurement license

shall be performed based on the license issued by the nuclear safety authority.

Scope of the license

1.3.4.0200. In the event that the specified conditions are fulfilled, the assembly license shall provide authorisation for the erection or installation of a system component, the connection thereof to other system components, the performance of preparatory activities necessary for commissioning, thus especially the flushing of pipelines and operation test of system components or the elements thereof.

1.3.4.0300. The license shall be of a fixed duration, which is determined by the nuclear safety authority taking into consideration the time necessary for the preparation and performance of the assembly, but it shall not exceed 5 years.

1.3.4.0400. The duration of the license may be extended once provided that the assembly is performed continuously and the legal provisions remain unchanged, for a period of no longer than 2 years.

Content requirements of the license application

1.3.4.0500. The license application shall contain:

- a) the name, safety and seismic safety classification of the system component, the name of the safety function performed by the system component;
- b) the task of the system component in each designed operation state of the nuclear power plant unit and the facility for the interim storage of spent fuel;
- c) the assembly-related technical and quality requirements specified in legislation and the Preliminary Safety Analysis Report of the nuclear facility with regard to the system component, the location of installation and the safety and seismic safety class, furthermore the provisions of the product designer;
- d) designer statements certifying the compliance with the nuclear safety requirements;
- e) design documentation of the assembly activity and technologies;
- f) documents certifying the audit plan of the assembly, the name of the audit body and its qualification to perform the activity;
- g) documentation containing the commissioning preparatory activities, thus especially pipeline flushing, energising, operation test, and the technical and administrative tasks to be performed in order to fulfil the nuclear safety requirements during the performance of the aforementioned activities;
- h) content requirements of the handover documentation certifying the compliance of the assembly;
- i) documents certifying the qualification of the organisation performing the design process and the assembly; and
- j) the reference number of the license issued by the nuclear safety authority in connection with the procedure, the name and identification numbers of documentation previously submitted by the licensee and used for the foundation of the application.

1.3.5. Operation license

1.3.5.0100. A license issued by the nuclear safety authority is necessary for the operation of pressure retaining equipment and pipelines categorised into safety classes 1, 2 and 3, subject to licensing by the nuclear safety authority.

Scope of license

1.3.5.0200. The operation license, if the conditions specified in it are fulfilled and following the successful, documented on- scene inspection of the pressure retaining equipment prior to the first commissioning performed by an inspection body authorised by legislation and independent of the licensee, shall provide authorisation to perform the programmes developed for the function and

commissioning tests of the equipment and after the successful performance thereof to operate the system component.

1.3.5.0300. The nuclear safety authority shall set the validity of the license taking into consideration the characteristics of the system component and the features and circumstances of the performance of the activities specified in the license, but it shall not be longer than the time period taken into consideration during design of the system component.

Content requirements of the license application

1.3.5.0400. The license application shall contain:

a) the name, safety and seismic safety classification of the system component, the name of the safety function performed by the system component;

b) the system component function in each designed operation state of the nuclear unit;

c) as-built documentation of the system component;

d) statement of the licensee that the as-built condition is in compliance with the following:

da) manufacturing or procurement licensing and the as-built design documentation,

db) technical and quality requirements specified in legislation and the Preliminary Safety Analysis Report of the nuclear facility with regard to the system component, location of installation, safety and seismic safety class, furthermore

dc) the provisions of the product designer;

e) name of the controlling body independent of the licensee, authorised by legislation which performs the on-scene inspection prior to the first commissioning and the inspection programme;

f) function tests prior to the commissioning programme, and demonstration that the function test or commissioning programme is suitable to verify the appropriate operation in line with the design of the system component;

g) schedule and frequency of the in-service inspections;

h) operational and maintenance instructions necessary for the safe operation of pressure retaining equipment and pipelines and the verification of the compliance thereof to achieve the objectives formulated in the aforementioned instructions;

i) operational limits and conditions specified in order to provide the safe operation of pressure retaining equipment and pipelines, furthermore the verification of safe operability under such limits and conditions;

j) documents certifying the existence of operating personnel of a staff number complying with the operational instructions and necessary for the safe operation, holding qualifications specified by legislation; and

k) the reference number of the license issued by the nuclear safety authority in association with the procedure, the name and identification numbers of documentation to substantiate the application, previously submitted by the licensee.

1.3.5.0500. The in-service inspection plan specified in Section 1.3.5.0400 g) shall contain the details of in-service and other inspections, furthermore the associated activities, in order to detect all anticipated or postulated failure in relation to the pressure retaining equipment characteristic to its operation at a time when the safety and soundness of the pressure retaining equipment has not yet been compromised by the failure.

1.4. NUCLEAR SAFETY AUTHORITY OVERSIGHT OF MODIFICATIONS

1.4.1.0100. The provisions of Section 1.4 shall be applied in the commissioning and operation lifecycle phases of the nuclear facility. The deviations from the contents of the licenses decided in the construction phase shall be managed according to the provisions in Section 1.10. In the termination lifecycle phase of nuclear facilities, the provisions of relevant licenses shall be applicable in the cases of technical configuration, organisational structure, control system, and the modification of technical and regulatory documents.

1.4.1.0200. The nuclear safety authority oversight of modifications is accomplished by instruments graded according to nuclear safety importance, but it shall extend over all technical, documentation and organisational modification, furthermore to temporary modifications. The instruments are graded according to the categorisation specified in Section 4.8.2.0800 of Annex 4, Sections 5.3.13.0300 to 5.3.13.0500 of Annex 5, and Sections 6.3.9.0700 to 6.3.9.0900 of Annex 6. The content and formal requirements of modifications subject to nuclear safety regulatory licensing obligation are independent of the designed continuance of the modified nuclear facility, system, structure or component. The nuclear safety authority shall not be influenced by the scheduled implementation date for the modification when determining the review method applicable to the substantiating documentation.

1.4.1.0210. In the case of nuclear facilities that belong under the effect of Annex 6 of Govt. Decree 190/2011. (IX. 19.) on physical protection requirements for nuclear facilities and the corresponding system of licensing, reporting and inspection, regarding the modifications of programmable systems of the facility a risk analysis shall be prepared for the protection of programmable systems.

Information regarding the agreed modifications, nuclear safety authority oversight plan

1.4.1.0300. A Modification Form detailing any planned modifications shall be completed by the licensee and forwarded to the nuclear safety authority for the planning and performance of its supervisory activity with regard to the proposed modifications as specified in Section 4.8.3.0300 of Annex 4, Section 5.3.13.0600 *b)* of Annex 5 and Section 6.3.9.1100 of Annex 6. An assessment of the preparation made for the modifications and an independent inspection report of the documents to be submitted, shall also be attached to the Modification Form, if these are required by the respective nuclear safety regulations related to the nuclear facility.

1.4.1.0400. Based on the Modification Form, the nuclear safety authority shall check whether the categorisation complies with the requirements.

1.4.1.0500. The nuclear safety authority shall prepare, maintain up-to-date, and execute the oversight plan for the modifications on the basis of the information submitted by the licensee in the Modification Form. Based on the supervisory plan, the nuclear safety authority – with the least possible hindrance to the normal activities of the licensee and suppliers – performs those inspections, which are necessary for the oversight of modifications in addition to licensing. In the oversight plan, the nuclear safety authority takes into consideration – amongst others based on the submitted documents which assess the preparation and execution of modifications – the experience collected during the preparation and execution of modifications carried out at the concerned licensee.

Licensing of modifications

1.4.1.0600. An modification license is necessary for the performance of modifications of categories 1 and 2. The Modification Substantiation Document specified in Section 4.8.3.0500 of Annex 4, Section 5.3.13.0600 *c)* of Annex 5 and Sections 6.3.9.1300 and 6.3.9.1400 of Annex 6 shall be submitted to substantiate the license application. The comprehensive safety assessment, documents of reviews performed by the licensee with the involvement of an independent technical expert and documents indicating the approval of the internal organisational unit of the licensee which supervises the modifications, shall be attached to the license application if it is required by the Nuclear Safety Code related to the nuclear facility. The schedule for the modifications shall also be attached.

1.4.1.0700. Regardless of the modification's categorization, a modification license shall be required for the implementation of modifications for safety classes 1 and 2 components in the nuclear facility which concern the following

a) the licensing pressure and temperature of pressure retaining equipment and pipeline,

b) opening and closing pressure and the release flow-rate of the pressure limiting system component, the number of pressure limiting valves, and the operational parameters of bursting discs, and

c) the in-service inspection frequency of pressure retaining equipment and pipelines, and the pressure value and temperature of the pressure test.

1.4.1.0800. The modification license shall authorise the licensee to perform the modifications. In the license, the nuclear safety authority shall approve the categorization, and determine at what time following the implementation of the modifications, the assessment of the modification shall be submitted for information, and furthermore, if necessary it shall oblige the licensee to obtain a modified operation license according to Section 1.2.5.0500. Different from Section 1.4.1.1200 or 1.4.1.1400, the nuclear safety authority may provide in the license that information regarding the performed activity according to Section 1.4.1.1200 or 1.4.1.1400 shall be sent in two parts, depending on the type of modification, prior to the start of commissioning, the implementation of an organisational change and to the introduction of a modified document. The license shall be valid until the date specified therein taking into consideration the characteristics of the modification, but for a maximum period of 5 years.

1.4.1.0900. In the case of nuclear power plant units, if the category of the technical modification reviewed in accordance with Section 4.8.3.1000 of Annex 4 upon completion of the implementation design becomes different from the result of categorisation specified in the Modification Form, then the modified Modification Form shall be sent to the nuclear safety authority. If a modification previously categorised into Category 1 or 2 falls into Category 3 after the new categorisation, then the licensee shall withdraw the modification license application, or submit an application to withdraw the issued modification license and, if a modification previously categorised into Category 3 falls into either classes 2 or 1 according to the new categorisation, then the licensee shall immediately compose a new Modification Substantiation Documentation, and submit a modification license application.

1.4.1.1000. Should the information included in the Modification Substantiation Documentation which is the basis for the modification license change during the preparation or execution of the modification, the licensee shall immediately inform the nuclear safety authority of these changes in writing, and request the withdrawal of the previous license and the issuance of a new license, if necessary.

Nuclear safety authority oversight of the modifications

1.4.1.1100. During the modification process, the nuclear safety authority – in accordance with the oversight plan – may perform ad hoc inspections, taking into consideration the nuclear safety importance of the modification. Based on inspection experience, the nuclear safety authority may order the preparation of

analyses and assessments, may oblige the licensee in a decision to modify or correct documents and activities, to perform further activities, or may forbid the performance thereof, if a breach of nuclear safety requirements is determined.

1.4.1.1200. In the case of licensed technical modifications, thirty days prior to the start of commissioning, the licensee of a nuclear facility, except for research reactors, shall send the Documentation Substantiating the Commencement of Post-modification Operation as specified in Section 4.8.3.1500 of Annex 4 and Section 6.3.9.2100 of Annex 6 to the nuclear safety authority for information. If, in the case of a licensed technical modification, no commissioning takes place, the nuclear safety authority shall designate the event in the license by taking into account the parameters of the technical modification specified in the application 30 days prior to which the Documentation Substantiating the Commencement of Post-modification Operation shall be submitted.

1.4.1.1300. Differing to Section 1.4.1.1200, if the nuclear safety authority requires in the modification license the provision of information in two parts, then the appointed parts of the Documentation Substantiating the Commencement of Post-modification Operation shall be submitted to the nuclear safety authority thirty days prior to the start of commissioning, whilst the remaining parts – if such parts shall be prepared according to the nuclear safety regulations related to the nuclear facility – shall be submitted to the nuclear safety authority with the independent expert review report, by the start of the last working day prior to commissioning.

1.4.1.1310. At the request of the licensee, the nuclear safety authority may also prescribe a period of less than 30 days in the license if the modification belongs to Category 2 under Section 4.8.2.0800, and the modification of the nuclear power plant unit is carried out

- a) during its main overhaul,
- b) in an extraordinary procedure, or
- c) in-service maintenance operating mode.

The period shall be determined by taking into account the complexity of the modification and the implementation schedule of the modification in such a way that it shall be sufficient for the review of the Documentation Substantiating the Commencement of Post-modification Operation.

1.4.1.1400. In the case of the licensed modification of the organisational structure, control system, and the technical and regulatory documents, the licensee of the nuclear facility different from the research reactor, shall send the summary as specified in clause 4.8.3.1600 of Annex 4, and clause 6.3.9.2200 of Annex 6, – should it need to be prepared according to the Nuclear Safety Code related to the nuclear facility – alongside the report on the independent review of

the document to the nuclear safety authority for information thirty days prior to the performance of modification.

1.4.1.1500. Differing to Section 1.4.1.1400, if the nuclear safety authority requires in the modification license the provision of information in two parts, then the appointed parts of the summary shall be submitted to the nuclear safety authority thirty days prior to the start of commissioning, whilst the remaining parts – should such parts need to be prepared according to the regulations related to the nuclear facility - shall be submitted to the nuclear safety authority alongside the independent expert review report, by the start of the last working day prior to the actual performance of the modification.

1.4.1.1510. In the case of the modification of the technical and regulating documents of the nuclear power plant, at the request of the licensee, the nuclear safety authority may also prescribe a period shorter than 30 days in the license if the licensing took place in an extraordinary procedure and the modification belongs to Category 2 under Section 4.8.2.0800. The period shall be determined by taking into account the complexity of the modification and the implementation schedule of the modifications in such a way that it shall be sufficient for the review of the summary description.

1.4.1.1600. In the case of the licensed technical modification of research reactors and spent fuel interim storage facilities, the Documentation Substantiating the Commencement of Post-modification Operation as specified in Section 5.3.13.0600 *d)* of Annex 5 and Section 6.3.9.0010. *d)* shall be sent to the nuclear safety authority for information at least 10 days before the start of commissioning. The summary description specified in Section 5.3.13.0600 *e)* of Annex 5 and Section 6.3.9.2200. of Annex 6 shall be sent for information at least 10 days prior to the performance of the licensed modification to research reactors' and spent fuel interim storage facilities' organisational structure and management system, and to technical and regulatory documents.

1.4.1.1700. The nuclear safety authority shall check the submitted Documentation Substantiating the Commencement of Post-modification Operation and the summary, also taking into consideration its supervisory plan, and shall perform ad hoc on-site inspections in the case of Category 1 technical modifications. If during the inspection of the submitted Documentation Substantiating the Commencement of Post-modification Operation or the summary description, or during the field inspection the nuclear safety authority finds such non-compliance, which endangers safe operation, then the nuclear safety authority shall withdraw the issued modification license or change the conditions by amending the license. In the case of such non-compliance revealed during inspection performed prior to issuing the license, the nuclear safety authority shall prohibit the continuation of the modification, and refuse to issue the license or shall determine conditions in the license.

1.4.1.1710. The nuclear safety authority shall take official note of the inspection of the documentation specified in Section 1.4.1.1700 and shall take record of the on-site inspections of Category 1 technical modifications. If the inspection of the Documentation Substantiating the Commencement of Post-modification Operation or the summary description did not find any non-conformity that jeopardises safe operability, the nuclear safety authority shall inform the licensee thereof in the note or the inspection record. The licensee shall continue the implementation of the modification in possession thereof.

1.4.1.1800. Unless otherwise required by the nuclear safety authority in the modification license, the Modification Assessment Report according to Section 4.8.4.0100 of Annex 4, Section 5.3.13.0600 *f*) of Annex 5, furthermore Section 6.3.9.2400 of Annex 6, shall be sent to the nuclear safety authority for information after 3 months following the execution of the modification, furthermore in cases where necessary, the operation license modification application shall be submitted. If no modification in the operation license of the nuclear facility is necessary, then final versions of those revised documents shall be attached to the Modification Assessment Report, which had not previously been sent to the nuclear safety authority and the previous versions of which were attached to the Documentation Substantiating the Commencement of Post-modification Operation.

1.5. LICENSING OF BUILDINGS, BUILDING STRUCTURES AND ELEVATORS

1.5.1. General provisions

1.5.1.0100. The list of buildings and building structures including the specification of their purpose shall be determined by the designer, and shall propose a classification of the special building based on the purpose. The list shall be included in the Safety Analysis Report of the nuclear facility. The list shall be first accepted by the nuclear safety authority in the construction license. The list shall be reviewed and updated as necessary periodically, but at least within the framework of the Periodic Safety Review .

1.5.1.0110. The licensee shall present the planned stages of the construction of the new nuclear power plant in the Preliminary Safety Analysis Report. It shall clearly mark the buildings belonging to the same construction stage and the buildings planned to be included into one building license application.

1.5.1.0200. A license under Sections 1.5.2 and 1.5.3 shall be necessary for buildings and building structures, except for the building structures specified in Section 52 *i*) of Annex 10:

- a*) if the construction process or the result thereof affects nuclear safety,
- b*) in the case of construction, extension and displacement of a building,

c) in the case of such renovation, restoration, modification, modernisation of a building or building structure, or change of its facade, where during the listed construction activity the support structure system or support structure elements of the building have to be changed, modified, demolished, reinforced, furthermore which causes a change in the number or function of separate building units, and

d) in the event of building demolition.

1.5.1.0300. If modifications are performed on buildings or building structures under commissioning, or in the case of operating nuclear facilities on buildings or building structures listed in Section 1.5.1.0100, then, in addition to the construction licensing procedure, the requirements specified in Section 1.4, Section 4.8 of Annex 4, Section 5.3.13 of Annex 5 and Section 6.3.9 of Annex 6 related to the concerned nuclear facility, shall also be applied.

1.5.1.0400. During decommissioning of nuclear facilities, the demolition works of buildings and building structures associated with the nuclear facility are licensed by issuance of a separate demolition license in a decommissioning licensing procedure according to Section 1.2.8.

1.5.1.0500. Unless otherwise required by the nuclear safety authority, upon completion of the works performed based on the construction license, an occupancy license shall be requested from the nuclear safety authority.

1.5.1.0510. If the building shall not be occupied without an occupancy license, the licensee shall apply for the occupancy license when the building becomes suitable for the intended and safe use.

1.5.1.0520. For the building sections, buildings and building structures constructed based on the same construction license, the occupancy license shall be simultaneously requested, except for those buildings and building structures, which are individually suitable for the intended and safe use. The occupancy license of those can be requested separately.

1.5.1.0600. If the construction works concern only a single part of the building, its other area may be used with the limitations specified in the construction license of the nuclear safety authority.

1.5.1.0700. In buildings subject to the Nuclear Safety Code, the license of the nuclear safety authority is necessary for the construction of elevators, their permanent installation into buildings and relocation, modification with changing their main technical data, occupancy and for demolition. The provisions of the decree on elevators, escalators and moving pavements, with the deviations specified in Section 1.5 shall be applicable for the regulatory licensing.

1.5.1.0800. In the case of elevators built into the containment (hermetical space) of the nuclear power plant and used only during maintenance of the unit:

a) the equipment can be occupied after comprehensive maintenance, calibration, and major inspection by the appointed organisation,

b) the technical safety reviews required during the operation period shall be performed with such frequency, as if the equipment were in continuous operation throughout the whole year, furthermore

c) following the performance of first line maintenance, the operational period shall be closed by the protocol of the equipment maintenance inspector.

1.5.1.0900. The nuclear safety authority, in its proceedings related to buildings, complies with the provisions of the act on modification and protection of the built environment containing the requirements for electronic administration via the electronic administration application operated by the nuclear safety authority.

1.5.1.1000. In the case of an emergency declared by the Government, the nuclear safety authority shall perform the site inspection of the buildings within the deadline determined based on the decision of the manager of the nuclear safety authority considering nuclear safety aspects.

1.5.1.1100. In the case of installing buildings within the impact area of a nuclear facility the rules and requirements for reporting and commenting according to settlement layout, and for building-technical design council commenting shall not be applied. In the case of installation outside the impact area, the comments shall be taken into account by providing priority to nuclear safety and security aspect.

1.5.1.1200. For the modification of the purpose of a building, building structure without construction works that requires licensing, the license of the nuclear safety authority is not required if, based on this decree, in relation to the modification, the modification license of the nuclear safety authority shall not be requested.

1.5.1.1300. The nuclear safety authority shall specify or extend the duration of the construction, occupancy license upon request or ex-officio after considering all issues within its competence.

1.5.2. Construction or demolition license of buildings and building structures

1.5.2.0100. The construction license shall be requested for the entire construction or demolition work to be performed. In the case of a construction accomplished in several phases, a separate construction license can be requested on a phase-by-phase basis for the buildings to be constructed in each individual phase, and for building parts suitable for normal and safe use.

Content requirements of the license application

1.5.2.0200. The construction license application shall contain:

a) verification of fulfilment and the technical basis of nuclear safety requirements,

b) quality management programme ensuring the appropriate performance of the activity,

c) licenses of other authorities associated with the procedure, and the documents substantiating them, and their summary assessments,

d) technical design documentation for construction licensing with the contents specified in Sections 1.5.2.0210 to 1.5.2.0230, and

e) independent technical expert opinion regarding the inspection and assessment of the documentation.

1.5.2.0210. Elements of the documentation under Section 1.5.2.0200 d) depending on the subject-matter of the design:

1. Technical descriptions: The descriptions, in addition to the technical information appearing in the technical drawings, shall describe the most important data of the design, especially the technical solutions meant to achieve nuclear safety.

1.1. Technical description for architecture:

It shall contain the technology to be installed in the building considering the characteristics of buildings, rooms and building structures of the facility in an extent sufficient for the licensing decision, adjusted to the function.

1.2. Building engineering technical description:

In addition to the relevant general provisions, it shall contain the relations of the building engineering and nuclear technology systems in an extent sufficient for the licensing decision.

1.3 Technical description of supporting structures:

It shall contain description of the design input data in a detail sufficient for the licensing decision, especially the special impacts derived from the design programme, technological requirements and the characteristics of the site as follows:

a) it shall include a description of initial data with effect on the design and construction of supporting structures, required for the implementation of buildings, with the level of elaboration required for substantiating the licensing decision, in particular, a description and the values taken into account of loads, effects and requirements arising from the design programme and the technological requirements, as well as from the earthquake threat to the site and extreme weather conditions, and shall specify the standards applied, and

b) on the basis of the structural calculations completed, it shall describe the system of supporting structures of buildings, the spans applied, the design of the main load-bearing components, their characteristic main dimensions, the quality and performance requirements for the materials and products included in the designs, including technological descriptions ensuring feasibility.

1.4. Technical description of building electrical systems: It shall describe electricity supply to buildings and the relations of the building electricity and nuclear technology systems.

It shall describe in detail the technology designed for the construction of the buildings.

1.5. List of layer sequences: It shall specify all different horizontal and vertical layer structures.

1.6. List of rooms: It shall specify the intended purpose, manageability and use categories, radiation conditions, floor areas and coverings of the individual rooms.

1.7. Description of relation with the technology systems, components in the building.

1.8. Safety and seismic safety classifications and their basis.

1.9. Description of the design basis of the safety classified building, building structure.

2. Design sheets: that comply with the related general prescriptions, professional requirements..

2.1. A site plan prepared by the designer using the map database of the real estate register of the State by identifying the building in subject, which shall include the border of the safety area in addition to the general requirements.

2.2. Clear identification of the building on the general plan of the facility.

2.3. Floor plans of different storeys.

2.4. Cross-sections in a number required for comprehension, but at least two, taken at an angle.

2.5. Facade plans depicting all characteristic outside views of the building.

2.6. Ground shaping plan.

2.7. Supporting structure design: it shall describe the geometry, material, characteristic dimensions and the connections of the supporting structures.

3. Calculations.

3.1. Architecture calculations

3.2. Supporting structure calculation: detailed statics calculation of the supporting structure that demonstrates the compliance with the special design requirements derived from the process and site characteristics.

3.3. Building energy calculations

3.4. In the case of safety classified buildings, the substantiation of capability to satisfy the design basis and fulfil safety function.

4. Quality management plan.

5. Justifications.

5.1. Designer statement.

5.2. Signature sheet.

It shall contain, in addition to the general requirements, the registration number of the HAEA according to the decree on the detailed rules of justification of applicability and registration and data content of the registration of professionals to pursue civil engineering-technical expert, civil engineering designer, technical building inspector and responsible construction supervisor activity in relation to structures applied for the use of atomic energy. The data and signature of the design inspector shall appear on the signature sheet.

5.3. The statistical sheet set forth in Annex 3 to Govt. Decree 312/2012 (XI.8.) on construction procedures and inspections, the procedures and inspections of the construction supervisory authority, and services provided by the construction authority.

5.4. Independent technical expert's report

6. Geotechnical documentation.

The geotechnical documentation shall be prepared according to the relevant rules, but shall contain at least:

6.1. Geotechnical category of the planned building and the basis of categorization.

6.2. The soil analysis report that describes the data obtained during the building geology data supply, field and laboratory examinations. It shall be described and justified that during the geotechnical survey the method, depth and distance of exploration and the quality of the samples were determined considering the geotechnical category of the planned building.

6.3. The geotechnical design that describes those measures and structures, that are suitable to solve the geotechnical issues, describes the calculations justifying the acceptability of them and reports those construction and technical oversight instructions that ensure the compliance with the design. In the geotechnical plan, if such solutions are intended to be applied, the diaphragm wall, pile foundation

and ground stabilization technology shall be described in detail. It shall be demonstrated that the relevant overload limit states will not occur in any construction condition. If there is an operating nuclear facility near the planned construction area, it shall be demonstrated that the soil and ground water conditions of the environment will not change for the impact of excavation to such an extent that it endangers other buildings. If a diaphragm wall is intended to be applied, then the specific requirements shall be described, with special attention to the tolerances, material properties, water tightness, connections of the diaphragm wall plates and the loads on the diaphragm wall structure.

7. Geodetic survey

1.5.2.0220. General specifications relating to the documentation under Section 1.5.2.0200 d):

1. An applicable standard in force or, in its absence, an unambiguous signal code specified by the Customer shall be applied for marking the given material or structure.

2.

3. The technical solution for precipitation drainage shall also be shown on the plot affected by construction activities if the construction activities also result in a change in the natural terrain conditions of the plot. If the natural terrain conditions of the plot is changed by fill or cut, the original and altered, final conditions of the plot shall be shown on a scaled cross-section of the terrain, depicting the elevations of the terrain.

4. In the case of construction activities carried out in more than one phase, the individual phases shall be clearly indicated on the design drawings.

5. Technical documentation for architecture shall be submitted as authenticated by the designer. The content of the documentation can be authenticated as a whole or by document parts.

6. With regard to the content requirements of the technical documentation for architecture, the relevant professional requirements of the Hungarian Chamber of Architects and the Hungarian Chamber of Engineers and the requirements of the design manual shall be taken into account regarding the level of the elaboration, contents and scales of the individual design sections of the documentation,.

1.5.2.0230. In connection with the documentation specified in Section 1.5.2.0200 d), requirements for the documentation required for requesting a position statement from the fire protection authority:

1. In the case of a building licensing procedure, contents of the fire protection documentation

1.1. The structure and contents of the architectural fire protection documentation, in addition to the provisions of this regulation, shall comply with the relevant health and fire protection regulations, and technical requirements. The fire protection chapters of the design documentation of the professional areas shall be harmonized to provide a conclusion on the complex fire safety of the facility.

1.2. The fire protection documentation prepared for the authority proceedings shall be developed by a fire protection expert or fire protection designer.

1.3 Annex: Declaration of the utility provider on the provision of firefighting water supply.

2.

1.5.2.0300. The recommendations related to the detailed content of the construction license application shall be included in a guideline.

1.5.2.0400. During the assessment of the construction license application the nuclear safety authority shall assess if the building and the related construction activity comply with the requirements of the relevant laws and standards and

a) if the planned location of the building is in compliance with the act on modification and protection of the built environment, and

b) if the preparation of the allotment hosting the building took place according to the requirements of the local construction regulations and the relevant laws.

1.5.3. Occupancy license of buildings and building structures

1.5.3.0100. Following construction, renovation, restoration, modification works and works necessary for use differing to the norm, performed on the building or building structure, when the building becomes suitable for normal and safe use, before occupancy, an occupancy license shall be requested from the nuclear safety authority.

1.5.3.0200. The occupancy license with regard to multiple buildings completed simultaneously on one site, or multiple differing types of construction works performed within one building, shall be requested jointly. In the case of constructions divided into multiple implementation phases, separate occupancy licenses may be requested on a phase-by-phase basis for buildings constructed in each individual phase, furthermore for building structures suitable for normal and safe use on their own.

1.5.3.0300. The duration of the occupancy license shall not be longer than the service life of the building section, building or building structure grounded in the design documents.

1.5.3.0400. The duration of the occupancy license may be extended in a separate procedure, if the technical compliance of the building is verified in the application by the end of the specific time period.

1.5.3.0500. The nuclear safety authority shall prohibit the occupation without license of buildings or building structures, or occupation different from the license.

Content requirements of the license application

1.5.3.0600. The following shall be attached to the occupancy license application:

a) list of documents which certify that the performed construction activity ensures fulfilment of the nuclear safety criteria,

b) those pages of the as-built design documentation, which indicate implementation differing from the construction license and the technical substantiation of the differences,

c) statements and certificates specified in the legislation on construction authority procedures and inspection,

d) statement of an independent technical expert that the quality assurance documents certifying the fulfilment of nuclear safety requirements related to construction and assembly works are fully comprehensive and genuine, furthermore that the building is suitable for normal and safe use, and

e) in the case of a change in the content of the Final Safety Analysis Report, the draft modification intended to be introduced when updating the Final Safety Analysis Report, unless otherwise required by the nuclear safety authority.

1.5.3.0700. The recommendations with regard to the detailed content of the occupancy license application and the application for occupancy license duration extension are included in a guideline.

1.5.3.0800. During assessment of the occupancy license application, the nuclear safety authority shall confirm on the site that

a) the construction work was performed in accordance with the construction license, the associated technical design documentation and the licensed deviations,

b) the building is suitable for its purpose specified in the construction license and is in a condition suitable for safe use, and

c) the statement of the responsible technical manager controlling the construction work is in conformity with the differences and their management method, with the fulfilment of the quality assurance programme and the completeness of the quality assurance documentation.

1.5.3.0900. During assessment of the application for extension of the duration of occupancy license, the nuclear safety authority shall confirm on the scene that

the building is in safe condition, suitable for its purpose and furthermore that the verification of technical compliance is based on real facts.

1.5.4. Construction and demolition license of elevators

1.5.4.0100. The license duration is specified by legislation on construction authority procedures and inspection.

Content requirements of license application

1.5.4.0200. In addition to the legal provisions on elevators, escalators and passenger conveyors, the license application shall contain the following:

a) name, identification sign and place of installation of the elevator,

b) purpose of the elevator and operation method (e.g. seasonal),

c) technical specification of the elevator, and

d) in the case of elevators located within the controlled zone of the nuclear power plant, the environmental features – thus especially temperature, humidity, potential radioactive contamination – and the compliance therewith.

1.5.5. Occupancy license of elevators

Content requirements of the license application

1.5.5.0100. In addition to the legal provisions on elevators, escalators and passenger conveyors, the license application shall contain the following:

a) name, identification number and place of installation of the elevator, and

b) copy of the protocol on the commissioning of the elevator.

1.5.5.0200. The recommendations for the building, usage and demolition licensing of elevators are set forth in guidelines.

1.6. NUCLEAR SAFETY AUTHORITY INSPECTION ACTIVITIES

1.6.1. General provisions

1.6.1.0100. The nuclear safety authority inspection activities shall be built on the comprehensive oversight system operated by the licensee, graded according to nuclear safety aspects.

1.6.1.0200. The nuclear safety authority may conduct inspection on nuclear safety related areas within the nuclear facilities, at licensees of nuclear facilities and their suppliers (hereinafter: subject of regulatory oversight) without time limit.

1.6.1.0300. The licensee shall ensure that the nuclear safety authority inspector has unimpeded access to the nuclear facility and the sites of suppliers at all times, furthermore to ensure the free movement of the nuclear safety authority inspector around the buildings and rooms of the nuclear facility or supplier site.

1.6.1.0400. The representative of the nuclear facility licensee shall be present at inspections carried out at the facilities of its suppliers.

1.6.1.0500. The representative of the subject of regulatory inspection having due professional competence and authorisation

- including the representative of the licensee in the case of a nuclear safety authority inspection of the suppliers – shall make every effort

- to cooperate and provide the available information – in order to facilitate a successful inspection.

1.6.1.0600. The nuclear safety authority inspector and the representative of the subject of regulatory inspection shall cooperate throughout the entire duration of the inspection, including the preparation of the protocol.

1.6.1.0700. The completed protocol shall be signed by the authority inspector and the representative of the subject of inspection, and in the case of inspections performed at the suppliers, the representative of the licensee.

1.6.1.0800. If as a result of the inspection, the nuclear safety authority inspector declares that the subject of regulatory inspection breach the provisions specified in its own regulation, or in the case of suppliers in the internal regulation of the licensee, the nuclear safety authority inspector shall draw the attention of the representative of the subject of regulatory inspection to this fact in the protocol, and by the establishment of a deadline call upon the representative – or the representative of the licensee in the case of inspections at the suppliers site – to restore the conditions specified for the safe application of atomic energy.

1.6.1.0900. The representative of the subject of regulatory inspection – if unauthorised or incompetent in the specific case – shall inform its authorised superior to make decisions and take measures regarding the call of the nuclear safety authority. The party subject of regulatory inspection shall comply with the call. If the caution is unheeded, the nuclear safety authority shall investigate the case, oblige the licensee in an *ex officio* procedure to restore the conditions specified for the safe application of atomic energy, and may at the same time conduct an enforcement procedure.

1.6.2. Purpose, key aspects and areas of the nuclear safety authority inspection

1.6.2.0100. The nuclear safety authority performs inspections, which takes account of also special inspection aspects arising from the given lifecycle phase of the nuclear facility. The areas of inspection are determined by the current lifecycle phase of the nuclear facility and the provisions of the effective legislation, whilst the frequency and the extent to which the inspection is detailed, is determined by the safety importance of the given area, and the experience of nuclear safety authority licensing, assessment and inspection.

1.6.2.0110. During the inspection, the licensee shall ensure that a representative authorized to make statements is available throughout the inspection to conduct the necessary interviews, describe the documents and perform walkdowns of the scenes.

Types of nuclear safety authority inspections

Comprehensive inspection

1.6.2.0200. A comprehensive inspection is performed on pre-specified areas of the licensee's activity by the nuclear safety authority. The purpose of the inspection is to examine the operation and interaction of entire processes.

1.6.2.0300. During the comprehensive inspection, the nuclear safety authority shall examine the operation of the nuclear facility, the operability of processes, fulfilment of controlling, supervisory and assessment tasks of the management from one or several complimentary aspects.

1.6.2.0400. The date and areas of inspection are specified in advance by the nuclear safety authority and the licensee is notified in due time in order that they have sufficient time to prepare for the comprehensive inspection.

1.6.2.0500.

1.6.2.0600. An assessment shall be made with regard to the experience of the inspection and forwarded to the licensee by the nuclear safety authority. Based on the observations of the nuclear safety authority assessment the licensee shall develop and execute an action plan. The nuclear safety authority shall be informed regarding the performance of tasks within the framework of regular reports.

Revealing inspection

1.6.2.0700. Should the nuclear safety authority experience detect deviation from the provisions or from good practice in connection with a part process, activity and event, the nuclear safety authority performs revealing inspections. Such inspections may be performed regarding event investigation.

1.6.2.0800. The nuclear safety authority shall determine the date and areas of inspection in advance and notify the subject of regulatory control regarding these details in adequate time in order that due time is available for the preparation of the revealing inspection.

1.6.2.0900. In order to successfully execute the inspection, the subject of regulatory inspection shall appoint and authorise a representative with the authorization to provide statements regarding the given topics.

1.6.2.1000. The subject of regulatory inspection shall develop and execute an action plan based on the inspection protocol. The licensee shall inform the nuclear safety authority regarding the performance of tasks in the form of regular reports.

Ad hoc inspection

1.6.2.1100. The nuclear safety authority shall perform ad hoc inspections in order to examine specific decision conditions, actions, deviations, information, states or locations. Ad hoc inspections may be announced in advance or may take place unannounced.

1.6.2.1200. In the case of inspections announced in advance, the nuclear safety authority, based on a programme or plan indicates the activity, action or location which it intends to inspect. Following this, the subject of regulatory inspection shall notify the nuclear safety authority in advance – a minimum of 24 hours prior – of the expected commencement time of the activity.

1.6.2.1300. In the case of previously unannounced inspections, at the beginning of the inspection, the nuclear safety authority inspector shall notify the employee of the subject of authority inspection appointed according to its internal regulation, of the fact of the inspection. The appointed employee shall arrange a representative who has authorisation to provide statements and sign the protocol in the areas of concern.

1.6.2.1400. The subject of regulatory inspection shall take actions according to the protocol, and the licensee shall report to the nuclear safety authority regarding the performance of tasks within the frame of regular reports.

Nuclear safety authority inspections associated with the design process and the specific lifecycle phases

Regulatory inspection of the design process

1.6.2.1500. During the design of a nuclear facility, the nuclear safety authority shall examine at least the following in addition to the specifications in Section 22(1):

a) operation of the quality management system of design, thus in particular, regulation of the licenses and liabilities of general designers, regulation of the design licenses and liabilities of specialist designers and suppliers, regulation of the execution of design modifications, regulation of the use of design software, codes and models;

b) development, approval and application of design manual;

c) appropriate usage of the unified marking of systems, structures and components;

d) regulation of marking of documentation and consistent application of the regulation;

e) compliance with configuration management requirements; and

f) execution of design modifications.

Regulatory inspection of site assessment

1.6.2.1600. During the site assessment, the nuclear safety authority shall examine at least the following aspects in addition to the specifications in Section 22(1):

- a)* methods applied during site survey and assessment; and
- b)* scope of implementation of ongoing and scheduled surveys during the site licensing procedure.

Regulatory inspection of construction

1.6.2.1700. During construction of nuclear facilities, the nuclear safety authority shall inspect the following aspects in addition to the specifications in Section 22(1):

a) manufacturing, construction and assembly of system components with safety classification, preparation activities necessary for commissioning, thus in particular (cleaning and flushing works, operational tests of active system components), furthermore performance of inactive function tests, which can be performed with fuel containing no nuclear material;

b) in the case of on-site construction and assembly works which cannot be or are difficult to examine, thus in particular foundations, insulations, and isolation system components;

c) activities with regard to main equipment, systems important to nuclear safety, especially the nuclear reactor, fuel storage parts, barriers preventing the release of radioactive material into the environment;

d) regulatory and power supply systems with safety classifications;

e) function tests of safety protective systems under inactive circumstances; and

f) training of the operational and maintenance personnel.

Regulatory inspection of commissioning 1.6.2.1800. During commissioning of nuclear facilities, the nuclear safety authority shall examine at least the following in addition to the specifications in Section 22(1), with the application of weighting according to safety:

a) results of four-party (designer, investor, contractor and operator) walk-down,

b) fulfilment of flushing criteria,

c) strength pressure test of pressure retaining equipment and pipeline systems,

d) mechanical tests of rotating equipment,

e) official tests of measurements,

f) official tests of controls,

g) locking tests,

h) termination of provisory required for commissioning, development of the final schemes,

i) notification of completion for completed construction works of the rooms, handover of rooms to the operator,

j) cold tests and pressure tests of the primary circuit,

k) revision I of primary circuit equipment,

l) warm tests and pressure tests of the primary circuit,

m) revision II of primary circuit equipment,

n) inspection prior to the occupancy licensing of buildings,

o) physical startup of units,

p) energetic startup of unit, and

q) results of tests performed during the loading of the unit.

Regulatory inspection of operation

1.6.2.1900. During the operation of nuclear facilities, the nuclear safety authority shall examine at least the following in addition to the specifications in Section 22(1):

a) operation,

b) management of operation,

c) activities intended to maintain the technical condition of systems, structures and components, such as maintenance and provision of its effectiveness, ageing management, environmental resistance qualification of equipment and the maintenance of the rated state,

d) management and usage of nuclear fuel,

e) selection and supervision of suppliers,

f) design,

g) procurement and manufacturing,

h) modifications,

i) provision of spare parts, communication, general order,

j) operation of training system,

k) nuclear safety regulatory licensing examination of employees,

l) management and independent reviews,

m) fulfilment of conditions and provisions specified in nuclear safety decision,

n) operational activities performed within the framework of periodic safety reviews, and

o) in-service inspection of pressure retaining equipment and pipelines subject to nuclear safety authority oversight performed by the inspection organization specified in Section 1.9.1.0700 with the contribution of the operator as required and the documentation thereof.

Regulatory oversight of termination

1.6.2.2000. During the termination of nuclear facilities, the nuclear safety authority shall examine the following in addition to the specifications in Section 22(1):

a) in the case of a delayed decommissioning strategy being selected, execution of condition maintenance and monitoring tasks,

b) the actual execution of decommissioning based on the Final Decommissioning Plan, Decommissioning Safety Analysis Report and the content of the decommissioning license,

c) update of the Decommissioning Safety Analysis Report, and

d) preparation for final disposal and exemption of radioactive wastes.

1.7. REPORTING OBLIGATION OF THE LICENSEE

1.7.1. General provisions

1.7.1.0100. In order to provide socially monitored use of atomic energy, the licensee shall prepare regular reports regarding nuclear safety related activities in each lifecycle phase of the nuclear facility, ad hoc reports regarding safety related events, furthermore reports assigned to condition during the construction of the nuclear facility, and submit these reports to the nuclear safety authority.

1.7.1.0200. The nuclear safety authority shall determine the scope of reporting obligation taking into consideration the risk from the operation of the nuclear facility and the type and technical features of the nuclear facility.

1.7.1.0300. The nuclear safety authority, in decisions made with regard to cases under its competence, may oblige the licensee to submit further reports in addition to those specified in Section 1.7.1.0100.

1.7.1.0400. A copy of reports submitted to the other authorities participating in the oversight of the nuclear facility, shall also be sent to the nuclear safety authority simultaneously with the submission.

1.7.1.0500. The licensee shall organise and by means of the appropriate method assess the data and information included in its reports, and where the assessment reveals any deficiency, the licensee shall formulate corrective actions.

1.7.2. Regular reports

1.7.2.0100. The nuclear safety authority shall utilise the submitted reports in its supervisory activity which includes the tracking and assessment of the condition of the nuclear facility, its systems, structures and components, the safety level of the nuclear facility and the activity of the licensee.

1.7.2.0200. The recommendations with regard to the content of regular reports of nuclear facilities and the possible fulfilment methods of the reporting obligation are included in a guideline.

Regular reports in the construction period of nuclear facilities

1.7.2.0300. The licensee shall annually submit an Annual Report, for the first case by the 31st of March of the year following the issuance of the construction license. Content requirements of the Annual Report:

a) assessment of the schedule in the construction license and in the previous report,

b) information with regard to the organisation and resources of the licensee,

c) assessment of the activity and safety performance of important suppliers,

d) justification for and compliance of important design modifications, deviation from the licenses, documentations supporting the licenses, and

e) the schedule of the forthcoming year.

Regular reports in the commissioning and operation periods of nuclear facilities

Regular reports of nuclear power plants

1.7.2.0400. The licensee shall submit the following regular reports about its activity in relation to the operation of nuclear power plant unit and nuclear safety to the nuclear safety authority:

a) Quarterly Report,

b) Annual Report, and

c) Periodic Safety Review Report

Regular reports of research reactors

1.7.2.0500. The licensee shall submit the following regular reports to the nuclear safety authority regarding the activities in relation to the operation of research reactors operated with periodic refuelling and with regard to nuclear safety:

a) Annual Report,

b) Preliminary Campaign Report,

c) Campaign Report,

d) Campaign Closing Report, and

e) Periodic Safety Review Report.

1.7.2.0600. The licensee shall submit the following regular reports to the nuclear safety authority regarding activities in relation to the operation of research reactors operated without refuelling and with regard to nuclear safety:

- a) Semi-annual Report, and
- b) Periodic Safety Review Report.

Regular report of the interim spent fuel storage facility

1.7.2.0700. The licensee shall submit the following regular reports regarding activities in relation to the operation of the interim storage facilities for spent fuel and nuclear safety to the nuclear safety authority:

- a) Semi-annual Report, and
- b) Periodic Safety Review Report

Content of regular reports and requirements for their submission

1.7.2.0800. The Quarterly Report shall provide detailed, organised and regular information to the nuclear safety authority with regard to:

- a) the operational features of the nuclear facility,
- b) activities and issues concerning operation and nuclear safety,
- c) nuclear safety related events occurring during operation and the execution of corrective actions agreed during their investigation and the fulfilment of nuclear safety authority provisions,
- d) the safety indicators of the relevant period,
- e) current operational issues and factors influencing safe operation,
- f) maintenance activity, and
- g) the own inspection activity of the licensee.

1.7.2.0900. The Semi-annual and Annual Report shall provide organised and regular information to the nuclear safety authority, summarising and assessing the activities in relation to the operation and nuclear safety of the nuclear facility.

1.7.2.1000. In the Preliminary Campaign Report, the nuclear safety authority shall receive preliminary information from the licensee concerning the scheduled refuelling in the research reactor and the technical and safety assessment of the following operational campaign.

1.7.2.1100. In the Campaign Report, the nuclear safety authority shall receive information following the refuelling performed in the research reactor, regarding the clarified technical and safety assessment of the campaign, based on the results and experience of the measurements and activities scheduled in the Preliminary Campaign report.

1.7.2.1200. In the Campaign Closing Report, the licensee shall provide summary and assessment information to the nuclear safety authority regarding the relevant campaign of the research reactor:

- a) operational features,
- b) activities and issues regarding operation and nuclear safety,
- c) nuclear safety related events occurring in the course of operation and execution of corrective actions agreed during the investigation thereof,
- d) fulfilment of nuclear safety authority provisions,
- e) safety indicators of the relevant period,
- f) current operational issues, and
- g) factors influencing safe operation.

1.7.2.1300. The licensee shall be obliged to submit reports to the nuclear safety authority as specified below:

- a) Quarterly Report by the 5th day of the second month following the quarter of the year,
- b) Semi-annual Report by the 5th day of the second month following the half year,
- c) Annual Report by the 31st of March of the following year,
- d) Preliminary Campaign Report 15 days prior to the commencement of the research reactor refuelling,
- e) Campaign Report within 30 days following the completion of the research reactor refuelling,
- f) Campaign Closing Report within 30 days following the completion of the relevant campaign of the research reactor.

1.7.3. Periodic Safety Review

1.7.3.0100. The licensees shall conduct a Periodic Safety Review every 10 years at all nuclear facilities and the results shall be presented to the nuclear safety authority in a Periodic Safety Review Report. If important new information about the safety of the nuclear facility, which requires development, arises between two Periodic Safety Reviews, such development shall be carried out forthwith.

1.7.3.0200. During the assessment the licensee shall analyse the consistency of the operation of the nuclear facility with the design basis, and all identified discrepancies relating to the content of the license shall be eliminated or nuclear authority approval shall be requested for persistence of any discrepancy.

1.7.3.0300. The assessment shall include the following:

a) identification of deviations from the nuclear safety code and internationally acknowledged good practices, the evaluation of the nuclear safety significance of the deviations, taking into account the operating experience and the results of science and technology,

b) the identification and evaluation of changes in the conditions of the nuclear facility, and its systems, structures and components,

c) regarding the plant site the identification and evaluation of new knowledge and facts from science results, technical development, and parameter monitoring, also

d) the identification and evaluation of deviations between previous results and the results of repeated analyses performed with new analysis equipment and methods.

1.7.3.0400. The scope of the assessment shall be clearly defined and substantiated. The scope shall be as extensive as practically achievable considering the nuclear safety aspects of the operating nuclear facility.

1.7.3.0500. The following shall at least be included in the scope of the periodic safety review:

a) the design of the nuclear facility documented in the Final Safety Analysis Report,

b) site characteristics, resistance to external hazard factors,

c) decommissioning,

d) the current condition of systems and system components,

e) equipment qualification,

f) ageing,

g) safety analyses,

h) analysis of hazard factors,

i) safety indicators of the nuclear facility,

j) evaluation and feedback of relevant technical and scientific results and operational experience,

k) utilization of research results and the experience of other similar nuclear facilities,

l) organisation, human factors, management system and safety culture,

m) procedures,

n) accident management,

o) nuclear emergency preparedness,

p) radiation protection of employees and the population and radiation exposure of the environment, and

q) experimental equipment in the case of research reactors,

r) in the case of a facility for the interim storage of spent fuel, evaluation and review of the compliance of the Preliminary Safety Analysis Report substantiating the extension activities, and

s) in the case of a nuclear power plant having more than one unit, possible interactions between the units.

1.7.3.0600. During the review modern, systematic and documented methodology shall be applied and both deterministic and probabilistic safety analyses shall be considered.

1.7.3.0700. All areas within the scope of the periodic safety review shall be reviewed and the identified discrepancies shall be compared to the licensing requirements and the current nuclear safety code and practice. The effects of identified discrepancies on safety shall be evaluated by appropriate methods. Any (positive and negative) discrepancies found shall also be evaluated comprehensively, and the reasonably feasible safety-enhancing measures shall be identified.

1.7.3.0800. The comprehensive evaluation of the nuclear safety of the nuclear facility shall be performed and based on the results of the review covering all areas it shall be presented that the nuclear facility is verifiably sufficiently safe for further operation at least until the next Periodic Safety Review. In the evaluation, the issues that may limit the safe operation of the facility shall be pointed out and the approach by which the licensee wishes to handle them shall be described.

1.7.3.0900. The licensee shall take all corrective actions resulting from the assessment, which are reasonably achievable and which are significant from nuclear safety point of view, within the shortest possible time, but not later than the commencement of the review. When the deadlines for implementation are set, the safety weight of the corrective actions shall also be taken into account.

1.7.3.1000. A guideline shall contain the recommendations of the detailed contents of the Periodic Safety Review Report.

1.7.4. Event reports

Event reports during the construction period of the nuclear facility

1.7.4.0100. The event reports of detection of significant failures and non-compliances during the design and construction works – including deviations in the management system that may result in non-compliances – shall be presented to the nuclear safety authority within 8 days of the detection. The event

investigation report shall be submitted within 60 days following the detection to the nuclear safety authority.

1.7.4.0200. The content requirements of the investigation report:

- a) the time and circumstances of the detection, the reporting person;
- b) the presentation of measures implemented to prevent the use of unsuitable product, service, process, location, labelling before the execution of corrective actions;
- c) description and safety evaluation of non-compliances;
- d) immediate actions performed by the initiator or others in order to mitigate the effects of the non-compliance;
- e) possibilities to improve the non-compliance;
- f) verification that the required safety margins exist;
- g) determination of design modifications necessary due to the non-compliance; also
- h) determination and deadlines of the necessary corrective and preventive measures.

Event reports during the commissioning, operation and decommissioning of the nuclear facility

The scope of reportable events

1.7.4.0300. The licensee shall submit an event report for all reportable events that occurred at the nuclear facility to the nuclear safety authority. The scope of reportable events shall be defined

- a) as part of the construction licensing procedure,
 - b) as part of the commissioning procedure,
 - c) as part of the operation licensing procedure, and
 - d) as part of the final termination and decommissioning licensing procedure
- in a decision issued by the nuclear safety authority.

If necessary, the nuclear safety authority may change the scope of reportable events at the conclusion of the Periodic Safety Review or through procedure initiated *ex officio*.

1.7.4.0400. A guideline shall contain the recommendations of the detailed contents of event reports.

INES classification, coordination and notification obligations

1.7.4.0500. All events shall receive INES classification. The licensee shall propose the rating and forward it to the nuclear safety authority via fax. The final classification shall be determined by the nuclear safety authority.

1.7.4.0600. The nuclear safety authority shall be notified of the INES classification of a reportable event within 16 hours, at the latest after the occurrence of the event, or if the event was detected later than it occurred after the detection of the event.

1.7.4.0700. The International Atomic Energy Agency shall be notified within 24 hours from the occurrence or detection of the event of events rated to Level 1 or higher. The notification is the responsibility of the nuclear safety authority. The licensee shall provide the necessary information and the English version of the INES event form to the nuclear safety authority within 20 hours following the event.

1.7.4.0800. The public shall be notified of events of the Level 1 or higher within 24 hours. Of events rated 0 or below the public shall be informed regularly. The licensee is responsible for the notification after consultation with the authority. The licensee shall submit the text of the INES Level 1 or higher event report to the nuclear safety authority within 20 hours from the occurrence or detection of the event, before the public is notified.

The method of fulfilling reporting obligations during commissioning, operation and termination

1.7.4.0900. The licensee shall fulfil its event reporting obligation according to the following:

a) a promptly reportable event shall be immediately reported by phone to the nuclear safety authority but not later than within 2 hours from its occurrence, or if the event was not detected at the time of occurrence within 2 hours from the detection,

b) an event that is not a promptly reportable event shall be reported by phone to the nuclear safety authority not later than within 14 hours from its occurrence or if the event was not detected at the time of occurrence within 14 hours from the detection,

c) INES classification shall be reported within 16 hours,

d) the event shall be reported in writing to the nuclear safety authority within 16 hours from the occurrence of the event,

e) the event investigation report shall be submitted to the nuclear safety authority within 45 days following the occurrence or detection of the event.

1.7.4.1000. The written report according to Section 1.7.4.0900 *d)* shall include the short description of the event, the developed operational conditions, the executed

and planned measures as well as their expected success and probable effects, and the preliminary safety assessment of the event.

1.7.4.1100. The deadline of the event investigation report according to Section 1.7.4.0900 e) may be extended by the nuclear safety authority based on a substantiated application.

The nuclear safety authority investigation and assessment of reportable events

1.7.4.1200. The nuclear safety authority shall assess the reported events based on the information available at the time of notification and it shall make a decision to:

a) investigate and evaluate the event based on the investigation report of the licensee,

b) investigate and evaluate the event based on the information continuously provided by the licensee and if necessary to perform on-scene inspection, or

c) investigate and evaluate the event independently of the investigation of the licensee in an on-scene nuclear safety authority investigation.

1.7.4.1300. During the on-scene investigation the nuclear safety authority may interview the involved personnel and their managers, may perform walk-downs, and request a reconstruction of the event sequence.

1.7.4.1400. The licensee shall ensure appropriate conditions and circumstances for the execution of the nuclear safety authority investigation. For this purpose it shall provide the available information related to the event and, if reasonably feasible and necessary, evidence shall be presented to the nuclear safety authority.

1.7.4.1500. The licensee together with the nuclear safety authority shall select events of which a report is to be prepared in the agreed form and content in Hungarian, English or both languages in order to promote information share among international nuclear safety authorities. The nuclear safety authority shall inform the licensee of the information received through international forums to ensure utilization of external experience.

1.7.5. Condition-based reports

1.7.5.0100. The licensee shall submit a condition-based report to the nuclear safety authority during the construction lifecycle phase 30 days prior to the following design and construction phases:

a) preparation of specifications of designer, construction, production and assembly contracts important to nuclear safety,

b) commencing construction of buildings of the nuclear island,

c) commencing the most important concrete works,

- d) lifting of main circulation loop equipment to place,
- e) commencing clean assembly works,
- f) commencing laying safety cables,
- g) commencing assembly of safety instrumentation and control systems, also
- h) commencing the commissioning of specific systems.

1.7.5.0200. The content requirements of condition-based reports:

- a) the evaluation of the status and development of design activities,
- b) the introduction and evaluation of the potential and selected supplier,
- c) the evaluation of the status of the design manual and the relevant chapters, also
- d) presentation of the quality management requirements of the commencing works.

1.7.6. Alert and notification in a nuclear emergency situation, natural or industrial catastrophe

1.7.6.0100. Following the development of a nuclear emergency situation, or the occurrence of an environmental or industrial catastrophe the licensee shall immediately or within 15 minutes perform the emergency classification. Within 30 minutes of the recognition of the emergency situation the relevant organisations of the national nuclear emergency response system shall be alerted. The alert shall be executed as defined in the Nuclear Emergency Preparedness and Response Plan of the nuclear facility. As an alert task the first notification shall be provided in writing within 60 minutes from the recognition of the nuclear emergency situation as determined by the Nuclear Emergency Preparedness and Response Plan of the nuclear facility about the known circumstances and consequences of the nuclear emergency situation.

1.7.6.0200. During the development of a nuclear emergency situation, or the occurrence of an environmental or industrial catastrophe, following completion of alerts tasks the licensee shall regularly inform the designated organizations of the national nuclear emergency response system. The notification reports on situation and technology shall be forwarded as the events of the emergency situation unfold but at least in intervals of 1.5 to 2 hours, or information shall be provided in any other way which equivalently allows for the independent evaluation of the nuclear emergency situation.

1.8. THE NUCLEAR SAFETY AUTHORITY EXAMINATION OF EMPLOYEES

1.8.1. Categorization of work positions based on their importance to nuclear safety

1.8.1.0100. The designer shall define with consideration to the specifics of the given nuclear facility the list of safety-important work positions and in the case of

nuclear power plants the list of safety-critical work positions. The list shall be included in the Preliminary Safety Analysis Report, preliminary version of the Final Safety Analysis Report and the Final Safety Analysis Report of the nuclear facility. The lists shall be approved by the nuclear safety authority in the construction license, and shall be reviewed periodically but at least during the periodical safety review and updated, if necessary.

1.8.2. Acquisition and renewal of authority licenses

1.8.2.0100. Safety-important work positions shall only be filled holding a nuclear safety authority license. To acquire a nuclear safety authority license a nuclear safety authority licensing exam shall be passed. The regulation of the exam developed by the licensee is authorized by the nuclear safety authority.

1.8.2.0200. The condition to acquire or renew the license is that the person fulfilling the work position

a) has the physical and psychological suitability that shall be verified by appropriate medical examination,

b) fulfils the required theoretical and practical training programme determined for the specific work position, and

c) is suitable to fulfil the tasks per the license by being in possession of the theoretical and practical knowledge required for the specific work position.

1.8.2.0300. The required level of theoretical and practical knowledge shall be confirmed with written and oral exam during the acquisition of the license, and with oral exam during the renewal of the license.

1.8.2.0400. The oral exam is successful if the representative of the nuclear safety authority considers the performance of the person taking the examination suitable for the work position.

1.8.2.0500. At the nuclear power plant for the acquisition and renewal of licenses in work positions of unit shift supervisor and control room personnel the suitable practical knowledge and competence shall also be verified with simulator testing. At the exam the candidate shall attest that is capable of observing and maintaining nuclear safety, and according to the candidate's authorization to intervention is able to:

a) work in a group to execute normal operational processes and to respond to DBC2-4 and DEC1-2,

b) evaluate the conditions of the nuclear power plant unit based on control room information, to define the deviations from normal operation and to execute the necessary actions, and

c) in the case of DBC2-4 and DEC1-2 to recognise and evaluate the situation, perform the required checks and actions, and to define further actions which are necessary to achieve and maintain safe condition of the nuclear unit.

1.8.2.0600. In the case of a new nuclear facility licenses shall be acquired until the commissioning license of the nuclear facility is granted.

1.8.2.0700. The licensee shall request the issuing and renewal of licenses. With the request 2 copies of documents certifying the fulfilment of license acquirement or renewal conditions shall be attached.

The validity of the license

1.8.2.0800. The time validity of the license is:

a) 3 years after the first licensing exam;

b) 3 years, if the examinee successfully passes the repeated exam after failing a renewal exam;

c) 5 years, if the result of the renewal exam is excellent;

d) Based on the result achieved during the licensing or renewal exam the duration of the license can be limited to 2 years or 1 year.

1.8.2.0900. The license loses its validity if:

a) it is not renewed within its validity period,

b) its owner fails to work in the licensed work position for a period of over 6 months,

c) the participation in the refresher training is not according to regulations, or

d) the license renewal exam is unsuccessful.

1.8.2.1000. The nuclear safety authority may revoke the license:

a) at the request of the license holder or the licensee, or

b) if regulations were severely violated and endangered the nuclear safety of the nuclear facility.

1.8.2.1100. The renewal conditions of a revoked license are defined for each specific case by the nuclear safety authority.

1.8.2.1200. In the case of a nuclear power plant with more of the same type of operating units, if the training programme of the nuclear facility covers the differences between the nuclear units and the regulation of work on other nuclear units is approved by the nuclear safety authority then the license of a specific work position may be issued with validity on all units of the nuclear power plant.

1.9. REQUIREMENTS FOR NUCLEAR SAFETY AUTHORITY OVERSIGHT ON PRESSURE RETAINING EQUIPMENT AND PIPELINES

1.9.1.0100. For pressure retaining equipment and pipelines under the effect of this regulation (related to Section 1.9. together: pressure retaining equipment and pipelines under nuclear safety authority oversight) the general authorisation, licensing, inspection and technical requirements for systems and system components of the Nuclear Safety Code shall be applied with adjustments defined under this Section.

1.9.1.0200. The following pressure retaining equipment and pipelines under nuclear safety authority oversight does not belong under the nuclear safety authority licensing procedure, if it is:

- a) a pipeline of ND <50 mm included in Safety Class 2 or lower,
- b) a pipeline of NP <20 bar included in Safety Class 3 or lower, or
- c) a vessel with a capacity of less than 0.1 m³ included in Safety Class 2 or 3.

1.9.1.0300. Unlike specified under Section 1.9.1.0200 in the case of a new nuclear facility pressure retaining equipment and pipelines under nuclear safety authority oversight do not belong under the nuclear safety authority licensing procedure, if:

- a) a pipeline is included in Safety Class 2 or 3 and ND <50 mm;
- b) a pipeline is included in Safety Class 3 and NP <20 bar; or
- c) a vessel is included in Safety Class 2 or 3 with a volume of less than 0.1 m³.

1.9.1.0400. Pressure retaining equipment and pipelines that do not belong under the nuclear safety authority licensing procedure shall be reported to the nuclear safety authority before their production or acquisition.

1.9.1.0500. A guideline shall contain the recommendations on the method and contents of the notification of pressure retaining equipment and pipelines that do not belong under nuclear authority licensing procedure.

1.9.1.0600. The nuclear safety authority may inspect the installation of pressure retaining equipment and pipelines that do not belong under the nuclear safety authority licensing procedure, furthermore the operational circumstances of pressure retaining equipment and pipelines that belong under the nuclear safety authority licensing procedure already operating and may order the licensee to provide the necessary safety measures.

1.9.1.0700. The post-manufacturing inspections of pressure retaining equipment and pipelines of the operating nuclear facility which belong under the nuclear safety authority licensing procedure, furthermore the in-service and special inspections according to the in-service inspection programme shall be the responsibility of the licensee. The licensee can involve an accredited inspection organization or accredited examination laboratory in the implementation of the programme.

1.9.1.0800. The inspection organisation and its employees shall be independent of effects that may influence the decision, and of those who have an interest in the results of the inspections.

1.9.1.0900. The inspection organisation shall possess appropriate equipment and employees to professionally execute the technical and administrative tasks of the inspection, thus shall be able to determine whether technical requirements had been fulfilled and to document the inspections.

1.9.1.1000. The availability of resources and competencies, the execution of pre-service, in-service and special inspections and the management of equipment passports shall be inspected by the nuclear safety authority.

1.9.1.1100. The in-service and special inspections of pressure retaining equipment and pipelines of a new nuclear facility under nuclear safety authority oversight shall be performed by an inspection organisation that is authorised by legislation and is independent of the licensee.

1.9.1.1200. Of the in-service examinations of pressure retaining equipment and pipelines operating at the nuclear power plant belonging under the nuclear safety authority licensing procedure the nuclear safety authority shall select which scheduled inservice inspections it wishes to inspect during the outage of the nuclear power plant unit and of this it shall inform the licensee before the commencement of the outage. Based on this notification the licensee shall inform the nuclear safety authority of the time of the inspection on the last workday before the scheduled inspection.

1.9.1.1300. In an operating nuclear facility the installation of new pressure retaining equipment and pipelines shall be considered a modification. The licensing of pressure retaining equipment and pipelines under nuclear safety authority oversight shall be executed according to modification regulations with regard to the requirements for pressure retaining equipment and pipelines determined in the Nuclear Safety Code for the specific nuclear facility.

1.9.1.1400. For the installation of pressure retaining equipment and pipelines under Section 1.9.1.1300, the designer shall define the method and scope of manufacturing inspections, the in-service inspection programme and inspection cycle considering the safety classification of the component. In-service inspections and cycle times shall be approved by the nuclear safety authority in the modification license, or in the case of a modification in Category 3 according to the procedure defined under Section 18(2).

1.9.1.1500. The nuclear safety authority in an authority procedure may prohibit the commissioning of a new pressure retaining equipment if

a) the manufacturing and the pre-service inspection had not been performed according to the plan,

b) the result of the pre-service inspection had been unsuccessful,

c) due to an irregular event that affected the pressure retaining equipment and was reported to the nuclear safety authority according to Section 1.7.4, the personnel, the population, and the environment is or may be endangered, or

d) the nuclear safety authority inspection discovered a deviation or failure that endangers the personnel, the population or the environment and this deviation or failure had not been terminated.

1.9.1.1600. For the oversight of new pressure retaining equipment the governing rules of operating pressure retaining equipment shall apply from the time of the first successful commissioning.

1.9.1.1700. The inspection following the modification or repair of pressure retaining equipment and pipelines under nuclear safety authority oversight shall be defined by the designer and it shall be included in the documentation of the modification or repair.

1.9.1.1800. The nuclear safety authority in an authority procedure may prohibit the commissioning of pressure retaining equipment and pipelines if

a) the nuclear safety authority inspection uncovered a deviation or failure that may endanger the personnel, population, or environment, and the deviation or failure was not eliminated,

b) it does not possess a valid and successful in-service inspection,

c) the modification or repair was not executed according to the plan and relevant technical documentation, or

d) personnel, population or the environment is or may be endangered due to an extraordinary event that affects the pressure retaining equipment and pipeline as reported to the nuclear safety authority according to Section 1.7.4.

1.9.1.1900. For pressure retaining equipment and pipelines belonging under the nuclear safety authority licensing procedure the modification of the cycle time of the inservice inspection shall be requested from the nuclear safety authority in special technically substantiated cases before the validity of the inspection expires, if:

a) it is verified that their technical condition ensures safe operability until the new date of the inspection, and the licensee performs those inspections and examinations that provide the necessary additional information, or

b) it is verified that their failure does not affect the personnel, public or environment and the licensee performs those inspections and examinations that provide the necessary additional information.

1.9.1.2000. Regarding the application the nuclear safety authority shall make its decision according to the procedure under Section 18(2).

1.9.1.2100. For pressure retaining equipment and pipelines that belong under the nuclear safety authority licensing procedure for special technically substantiated cases it may be requested that the periodical inspection method may be substituted with another inspection method while complying with the same safety requirements. Regarding the request the nuclear safety authority shall make its decision according to the procedure under Section 18(2).

1.10. NUCLEAR SAFETY OVERSIGHT OF DEVIATIONS FROM THE PROVISIONS OF THE LICENCES DURING CONSTRUCTION

1.10.1.0100. Requirements of Section 1.10. shall be applied in the construction licensing phase of the nuclear facility.

1.10.1.0200. If, after becoming final, the Licensee plans a deviation from the construction, building, commissioning, fabrication, purchase or assembly license issued in relation to a new nuclear facility or from the documentation supporting the license according to Section 9.2.1.1300., the Licensee shall notify the nuclear safety authority of the deviation.

1.10.1.0300. Together with the notification the Licensee shall send a preliminary safety assessment according to Section 9.2.1.1300. and the results of the categorization per Section 9.2.1.1400. including a justification.

1.10.1.0400. The authority shall assess the documents submitted by the Licensee based on Section 1.10.1.0300., and depending on its results it approves or modifies the category determined by the Licensee. The Licensee shall be informed of the decision.

1.10.1.0500. Depending on the category determined or approved by the authority, in order to have the deviation approved, the Licensee

a) shall submit an application for approval of the deviation in the case of a category per Subsection 9.2.1.1400. a), or

b) shall follow the provisions of Section 9.2.1.1100. on the documented procedure in the case of a category per Subsection 9.2.1.1400. b).

1.10.1.0600. In the case per Section 1.10.1.0500., using the content and formal aspects of the original license application the Licensee shall submit to the authority a documentation compiled as follows:

a) it shall determine the scope of deviating documents, and describe the content, extent and reason of the deviation from the documentation, and the circumstances that had led to the deviation;

b) it shall demonstrate the technical and safety compliance of the planned deviation, and compliance with the nuclear safety requirements;

c) based on Subsections a) and b) it shall modify the documents supporting the license application per Section 1.10.1.0200. and concerned by the deviation.

1.10.1.0700. In the cases according to Subsection 1.10.1.0500. b) after approval by the authority in line with Section 1.10.1.0400. the Licensee shall describe the deviations in the Annual Report per Section 1.7.2.0300.

1.10.1.0800. The modifications, changes decided during the construction period shall also be described in the documentation supporting the commissioning license application with the detail as the given issue appeared in the construction license documentation.

1.11 DEMONSTRATION OF CAPABILITIES OF CONTRACTORS

1.11.1.0100. The licensee shall inform the nuclear safety authority on the conduct of the qualification audit related to the demonstration of capabilities of the contractors with or without a site audit. In case of an inland audit the notification shall be sent 10 working days before the audit, while in case of an audit abroad 20 days before it.

1.11.1.0200.- If the nuclear safety authority informs the licensee on its intention to participate in the conduct of the site audit, the licensee shall send the relevant internal regulations of the contractor to the authority 5 days before the date of the site audit.

1.11.1.0300. A 15 days within the completion of the qualification audit(s) the audit report and its annexes (especially qualification sheet, annexes of the qualification sheet, sheet describing the qualification process, records of deviations, participation list and all further documents generated in relation to the procedure) shall be sent to the nuclear safety authority or shall be made accessible electronically.